



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-295

ZION NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 154
License No. DPR-39

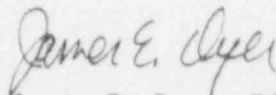
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 4, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-39 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 154, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 2, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-304

ZION NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-48

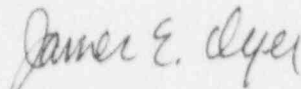
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 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 4, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-48 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 2, 1994

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE NO. DPR-39

AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-48

DOCKET NOS. 50-295 AND 50-304

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

Remove Pages

156

160

163

Insert Pages

156

160

163

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>3.7 <u>STEAM GENERATOR EMERGENCY HEAT REMOVAL</u></p> <p><u>Applicability:</u></p> <p>Applies to auxiliary feedwater pump system, auxiliary feedwater supply system, and steam generator safety valves.</p> <p><u>Objective:</u></p> <p>To insure adequate plant cooldown capabilities upon loss of normal feedwater flow and loss of main condenser vacuum.</p> <p><u>Specification:</u></p> <p>3.7.1 Steam Line Safety Valves per unit</p> <p>A. Twenty ASME code safety valves (5 per steam generator) shall be OPERABLE whenever the reactor is heated above 350°F except as specified in 3.7.1.C, 3.7.1.D, and 3.7.1.E.</p> <p>B. Deleted</p>	<p>4.7 <u>STEAM GENERATOR EMERGENCY HEAT REMOVAL</u></p> <p><u>Applicability:</u></p> <p>Applies to surveillance of auxiliary feedwater pump system, auxiliary feedwater supply system and steam generator safety valves.</p> <p><u>Objective:</u></p> <p>To insure availability of the above system and valves.</p> <p><u>Specification:</u></p> <p>1. Steam Line Safety Valves per unit</p> <p>A. The Steam Generator Safety Valve set pressures, specified in Table 4.7-1, shall be verified pursuant to Specification 4.0.5. Following testing, the set pressure of the valves tested shall be left within $\pm 1\%$ of the values specified in Table 4.7-1.</p> <p>B. Deleted</p>

<u>1A(2A)</u>	Steam Generator Safety Valves			Set Pressure (± 3%)	Orifice Size	Steam Flow (LB/hr)
	<u>1C(2C)</u>	<u>1D(2D)</u>	<u>1B(2B)</u>			
MS0014	MS0019	MS0024	MS0029	1050 psig	Q	584,105
MS0015	MS0020	MS0025	MS0030	1063 psig	Q	584,105
MS0016	MS0021	MS0026	MS0031	1076 psig	R	845,763
MS0017	MS0022	MS0027	MS0032	1088 psig	R	845,763
MS0018	MS0023	MS0028	MS0033	1100 psig	R	845,763

Steam Generator Safety Valves, Set Pressures, Orifice Sizes, and Steam Flows

TABLE 4.7-1

Bases:

- 4.7 The testing of the Steam Generator Safety Valves and Auxiliary Feedwater Pumps pursuant to Specification 4.0.5 ensures equipment availability by testing in accordance with the ASME Inservice Testing Plan. The ASME Code establishes industry standard methods, intervals, and record requirements for testing, as well as guidelines for the evaluation of test results. Although Table 4.7-1 allows a $\pm 3\%$ set pressure tolerance for the OPERABILITY of the Steam Generator Safety Valves, each valve tested is reset to $\pm 1\%$ during surveillance testing to allow for setpoint drift.

The four hour delay in the surveillance and testing of the turbine driven auxiliary feed-water pump until the reactor has reached the hot standby condition is to prevent unnecessary cooldown of the reactor coolant system during periods when the reactor is not available as a heat source.