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MICHIGAN'S PROGRESS

Big Rock Point Nuclear Plant, 10260 US-31 North, Charlevoix, MI 49720

William L Beckman
Plant Manager

February 1, 1991

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of January 1, 1991 through January 31, 1991.

Sincerely,

W L Beckman
Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission
DRHahn, Department of Public Health
RCallen, Michigan Public Service Commission
SHall, Michigan Department of Labor
PDKrippner, American Nuclear Insurers
INPO Record Center
NRC Resident Inspector
Document Control, Big Rock Point, 740/22*35*10
DPHoffman, P24-117B
KWBerry, P24-614B
File

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NUCLEAR OPERATIONS DEPARTMENT
Unit Shutdowns and Power Reductions

Report Month January 1991	Docket Number SS - 184	Unit B1, Cook Point Plant	Date February 1, 1991	Compiled by JL Johnston	Telephone (419) 341-4007 ext 220
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Number	Date	Type ¹	Duration (hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report Number	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
							W067		

¹F = Forced
S = Scheduled

²Reason:
A = Equipment Failure (Explain)
B = Maintenance of Test
C = Refueling
D = Regulatory Restriction
E = Operator Training and License Examination
F = Administrative
G = Operational Error (Explain)
H = Other (Explain)

³Method:
1 = Manual
2 = Manual Scram
3 = Automatic Scram
4 = Other (Explain)

⁴Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

BUCKET NO. 50-155 DATE: 2 / 1 / 91
 BY: JR JOHNSON
 PHONE: 615-547-6537, EXT 223

NOTES:

- OPERATING STATUS
1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT
 2. REPORTING PERIOD: 1 / 91
 3. LICENSED THERMAL POWER (MWt): 243
 4. NAMEPLATE RATING (GROSS MWe): 75
 5. DESIGN ELECTRICAL RATING (NET MWe): 71
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 71.0
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 67.0
 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:
 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
 10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	744.0	244099.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.0	744.0	176652.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	744.0	744.0	173751.6
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MMH)	164016.0	164016.0	33129193.0
17. GROSS ELECTRICAL ENERGY GENERATED (MMH)	52002.0	52002.0	10518646.0
18. NET ELECTRICAL ENERGY GENERATED (MMH)	49365.5	49365.5	9749420.2
19. UNIT SERVICE FACTOR	100.0%	100.0%	71.2%
20. UNIT AVAILABILITY FACTOR	100.0%	100.0%	71.2%
21. UNIT CAPACITY FACTOR (%SIN) (GCR NET)	99.0%	99.0%	60.5%
22. UNIT CAPACITY FACTOR (%SIN) (GCR NET)	92.2%	92.2%	56.6%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	12.1%

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH):

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

DAY	AVERAGE DAILY POWER (1WT)	(MWEN)	(1/91) - CYCLE 25
1	217.83	65.52	
2	219.37	66.03	
3	219.17	65.91	
4	219.96	66.19	
5	219.25	66.00	
6	218.83	65.84	
7	218.33	65.75	
8	217.29	65.52	
9	215.67	65.26	
10	220.62	66.37	
11	219.29	65.88	
12	218.96	65.93	
13	218.83	65.85	
14	221.12	66.72	
15	220.79	66.76	
16	222.17	66.64	
17	221.04	66.51	
18	218.79	66.25	
19	219.50	66.11	
20	223.21	67.17	
21	220.92	66.65	
22	221.75	66.50	
23	220.29	66.54	
24	220.00	66.31	
25	223.79	66.76	
26	222.96	67.02	
27	222.37	66.90	
28	222.25	66.77	
29	221.21	66.64	
30	225.04	67.63	
31	223.37	67.35	

Refueling Information Request

1. Facility name: Big Rock Point Plant
2. Scheduled date for next refueling shutdown: November, 1991
3. Scheduled date for restart following shutdown: January, 1992
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? No

If no review has taken place, when is it scheduled?

The PRC review will be completed before reload commences.

5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
7. Number of fuel assemblies in: core 84; spent fuel storage pool 274; new fuel storage 0.
8. Present licensed spent fuel storage capacity: 441
Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: 1995