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10 CFR Part 50
Section 50.90

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Supplemental Information for
License Amendment Request Dated September 13, 1990
Addition of Reference to Approved Methodology

Attached are two items requested by the NRC staff to support the license amendment request dated September 13, 1990. The first item is an example large break loss of coolant accident calculation using the revised methodology referenced in Section 6 of the proposed Technical Specification change. This calculation uses an $F_0 = 2.4$. Calculations are currently being performed to increase this value of F_0 . The second item is a discussion of the acceptance criteria for the steady state calculation results.

Please contact us if you have any questions related to this request.

Thomas M Parker
Manager
Nuclear Support Services

c: Regional Administrator, Region III, NRC
Senior Resident Inspector, NRC
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MPCA

Attn: J W Ferman
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Attachments:

Revised Prairie Island Large Break LOCA Calculation Using WCAP-10924
Volume 1 Addendum 4 Model Changes

Core Flow Difference Observed in WCOBRA/TRAC Calculations

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