

EDISON DRIVE AUGUSTA, MAINE 04336 (207) 623-3521

November 30, 1982 MN-82-241

JHG-82-227

United States Nuclear Regulatory Commission Washington, D.C. 20555

Attention: Mr. Robert A. Clark, Chief

Operating Reactors Branch 3

Division of Licensing

Office of Nuclear Reactor Regulation

References:

(a) License No. DPR-36 (Docket No. 50-309)

(b) USNRC Letter to MYAPCo, dated March 17, 1982

(c) USNRC Letter to MYAPCo, dated May 5, 1982

(d) MYAPCo Letter to USNRC, dated June 3, 1982, MN-82-108
(e) MYAPCo Letter to USNRC, dated June 25, 1982, MN-82-122
(f) MYAPCo Letter to USNRC, dated June 30, 1982, MN-82-124

Subject: Post-TMI Requirements (Generic Letters No. 82-05 and No. 82-10)

Dear Sir:

Enclosed are updates on selected TMI action items, most of which were scheduled for completion during the 1982 refueling outage.

We trust you will find this information satisfactory; however, if you require further clarification, please contact us.

Very truly yours,

MAINE YANKEE ATOMIC POWER COMPANY

John H. Garrity, Senior Director Nuclear Engineering & Licensing

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Enclosures (2 pages)

cc: Mr. Ronald C. Haynes Mr. Paul A. Swetland A046

ENCLOSURE TO MYAPOO LETTER TO USNRC DATED NOVEMBER 30, 1982

II.B.2 Plant Shielding

One item remains to be completed which is the installation of reach rods on two valves required for residual heat removal system operation. We indicated that this modification would be addressed by the end of the 1982 refueling outage; however, because of ALARA considerations, this modification has been delayed until after plant startup. This modification will be completed by the end of 1982.

II.B.3 Post Accident Sampling Capability

 Coolant Sampling - System installation was completed April 1, 1982, and the two sensors returned to the manufacturer have been repaired and installed. This system is operable with the exception of completing final testing and calibration of the gas collection portion of the system to perfect sampling techniques.

This testing cannot be completed until after plant startup when reactor coolant gas activity levels will rise to those necessary for final calibration.

II.E.1.2 Auxiliary Feedwater Initiation and Flow Indication

 Initiation-The safety grade initiation system logic was installed and operable as of January 1, 1982. During the 1982 refueling outage, the AFW flow isolation valves were upgraded to safety grade. Redundant flow isolation valves have been deleted from this design change. This is a change from the plan described in Reference (d).

II.F.1 Accident Monitoring

 Noble Gas Monitor (Ventilation Stack) and Main Steam Line Monitors -Rubber mounts to correct the vibration problem on the main steam line monitors have been installed. Following plant startup from the current outage, we will determine whether the vibration problem has been corrected.

Numerous problems have been encountered during calibration caused by defective components in these monitors.

The manufacturer's technical representative was not able to identify and repair all defective components to allow the calibration of these monitors. We are continuing in our efforts to trouble shoot and repair these monitors so that they can be calibrated and declared operable.

6. Hydrogen Monitor - Qualified hydrogen monitors are installed and are considered operational for on-line hydrogen monitoring. A change to the grab sample arrangement is being studied to improve the ease of collecting samples for off-line analysis. - 2 -

II.D.1.2 RV and SV Test Programs

The status of this program was reported in Reference (f).

Information on PORV operability was contained in CEN-213 - "Summary Report on the Operability of Power Operated Relief Valves in C-E Designed Plants." This report was previously forwarded to the NRC by the C-E Owner's Group. Maine Yankee considers Report CEN-213 to provide the required information on PORV operability for Maine Yankee.

A similar report is being developed by the C-E Owner's Group which will address safety valve operability for Maine Yankee. Our intent is to provide you with a partial report by January 1, 1983. A final report which will serve as our final submittal will be provided by April 1, 1983.

Additional information on plant specific piping evaluation will be provided by January 1, 1983. This will be followed by a final report by April 1, 1983.

II.D.1.3 Block Valve Test Program

Information on block valve operability was provided in Reference (f). In summary, based on the successful results of both the EPRI tests on similar valves and the successful in-situ tests at Maine Yankee, we feel that the operability of the PORV block valves as installed at Maine Yankee has been clearly demonstrated.

III.D.3.4 Control Room Habitability

The control room breathing air supply was automated during the 1982 refueling outage.