Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st. Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

February 15, 1994

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Licensee Event Report #94-002-00, Docket #050-373 is being submitted to your office in accordance with 10CFR50.73(a)(2)(iv).

D. J. Ray
Station Manager
LaSalle County Station

DJR/JR/mkl

Enclosure

xc: Nuclear Licensing Administrator
NRC Resident Inspector
NRC Region III Administrator
INPO - Records Center
IDNS Resident Inspector

IE22
1/1

LICENSEE EVENT REPORT (LER)

Form Rev 3.0

Facility Name (1)	Docket Number (2)	Page (3)
LaSalle County Station Unit 1	0 5 0 0 0 3 7 3	1 of 0 3

Title (4)

Single Control Rod Scram Due To Bad Fuse Clip

Event Date (5)			LER Number (6)			Report Date (7)				Other Facilities Involved (8)	
Month	Day	Year	Year	/// Sequential Number ///	Revision Number	Month	Day	Year		Facility Names	Docket Number(s)
0 1	2 2	9 4	9 4	--- 0 0 2 ---	0 0	0 2	1 5	9 4			

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)									
POWER LEVEL (10)	0 7 4		20.402(b)		20.405(c)	X	50.73(a)(2)(iv)		73.71(b)		
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)		
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		Other (Specify		
			20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		in Abstract		
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		below and in		
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		Text)		

LICENSEE CONTACT FOR THIS LER (12)

Name	TELEPHONE NUMBER
	AREA CODE
John Reimer, System Engineer, Extension 2249	8 1 5 3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	///	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	///
X	A A	F U B	G 0 8 2	N	///						///
					///						///

SUPPLEMENTAL REPORT EXPECTED (14)		Expected Submission Date (15)	Month	Day	Year
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e, approximately fifteen single-space typewritten lines) (16)

On January 22, 1994 during half scram testing (LOS-RP-M1) on Unit 1 with power at 74%, control rod 54-47 (5A-peripheral rod) scrammed to the full in position. The unit was dropped 75 MWE per LOA-RD-06, "Control Rod Drift/Individual Rod Scram". No thermal limits were exceeded with this rod being at the full in position. A Qualified Nuclear Engineer (QNE) was notified that the rod was at the full in position and the QNE stated no further action was required for the scrammed rod. The event was reported to the NRC per 10CFR50.72(b)(2)(ii) within the required four hours.

Investigation revealed the "B" RPS fuse was loose in its clip. This coupled with the half scram from "A" Reactor Protection System (RPS) from the performance of LOS-RP-M1 caused the rod to scram in.

A work Request L26894 was written and the control rod was removed from service. The fuse block was found cracked and subsequently satisfactorily replaced. Since this work does not affect the scram time of the control rod, no additional tests were required on the control rod prior to returning the rod to service.

A new parts evaluation (1-94-0014) was performed and concluded that the electrical box assembly supplied by General Electric (part # 922d234G001) which contains the fuse holder on a terminal board is not safety related. Any failure of the terminal board would result in a control rod scram. It is the first time that this particular part had to be replaced at the station. This is considered an isolated event and no further action is required.

This event is reportable pursuant to 10CFR50.73(a)(2)(iv) due to an actuation of an Engineered Safety Feature ESF.

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as (XX).

A. CONDITION PRIOR TO EVENT

Unit(s): 1 Event Date: 1/22/94 Event Time: 0430 Hours

Reactor Mode(s): 1 Modes(s) Name: RUN Power Level(s): 74%

B. DESCRIPTION OF EVENT

On January 22, 1994 Unit 1 was in Operational Condition 1 (Run) at 74% power. At 0430 hours during half scram testing (LOS-RP-M1) control rod 54-47 (5A-peripheral rod) scrammed to the full in position. Reactor power was reduced 75 MWE per Operating Procedure LOA-RD-06, "Control Rod Drift/Individual Rod Scram" (RD) [AA].

No thermal limits were exceeded with this rod being at the full in position. A Qualified Nuclear Engineer (QNE) was notified that the rod was at the full in position and it was determined that no further action was required for the scrammed rod.
The event is reportable per 10CFR50.73(a)(2)(iv) due to an automatic Engineered Safety Feature (ESF) actuation.

C. APPARENT CAUSE OF EVENT

Investigation revealed the "B" Reactor Protection System (RPS, RP) [JC] fuse was loose in its clip. This coupled with the half scram from "A" RPS from the performance of LOS-RP-M1 caused the rod to scram in.

D. SAFETY ANALYSIS OF EVENT

Appropriate actions were taken when the rod scrammed in and no additional actions were necessary. No thermal limits were exceeded with the peripheral rod at the full in position. Actions provided in LOA-RD-06 provide conservative actions for any single rod scram.

E. CORRECTIVE ACTIONS

1. Work Request L26894 was written and the control rod was removed from service. The fuse block was found cracked and subsequently satisfactorily replaced. Since this work does not affect the scram time of the control rod, no additional tests were required on the control rod prior to returning the rod to service.

2. A new parts evaluation (1-94-0014) was performed and concluded that the electrical box assembly supplied by General Electric (part # 922d234G001) which contains the fuse holder on a terminal board is not safety related. Any failure of the terminal board would result in a control rod scram. It is the first time that this particular part had to be replaced at the station.

3. A programmatic review of fuse/fuseholder issues is being performed to determine if further action is necessary to address this type of event.

F. PREVIOUS EVENTS

A review of all single rod SCRAMS that occurred during the past three years determined that none were caused by a problem with fuse clips.

G. COMPONENT FAILURE DATA

Manufacturer	Nomenclature	Model Number	MFG Part Number
General Electric	Electric Box Assembly		922d234G001

373-180-94-00144 PIF

SHEET 1

EVENT SUMMARY AND CAUSE CODES

DVR Number __-_-__-___

- [ ] Lost generation
- [ ] Cost > $25,000
- [ ] Hazard or Spill
- [ ] Personnel injury

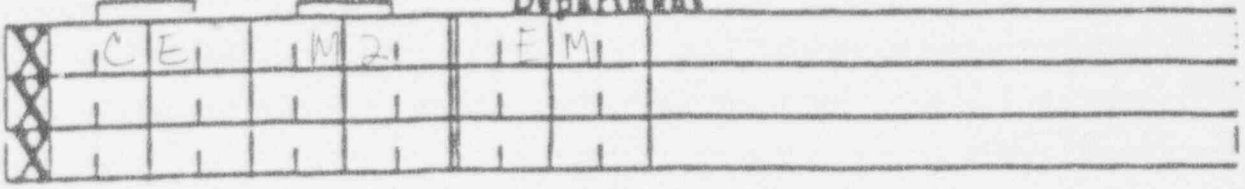
Component type

- [ ] Reactor trip
- [ ] ESF actuation
- [ ] NRC reportable
- [x] LER
- [ ] PSE

Failure mode

- [ ] NRC violation, level__
- [ ] GSEP event, class____
- [ ] Tech Spec LCO
- [ ] Potential or future lo
- [ ] SALP functional area_

Department



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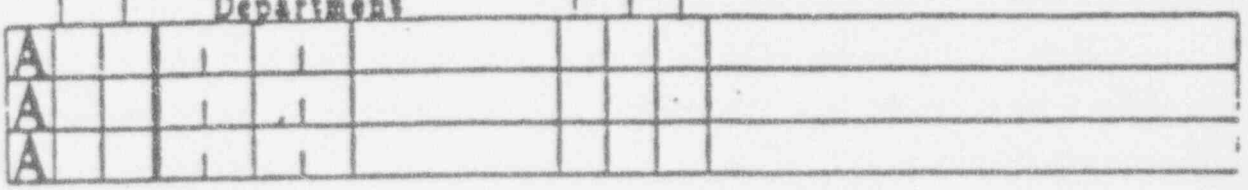

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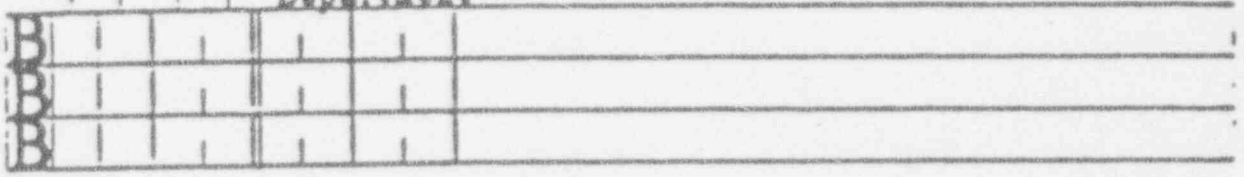

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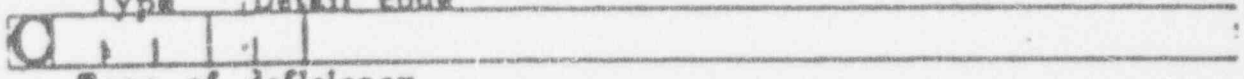

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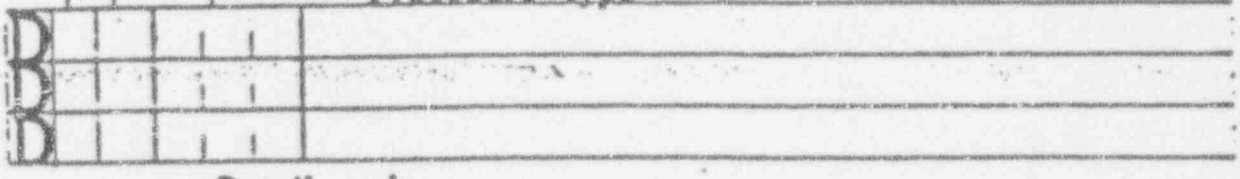

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