Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

February 15, 1994
LIC-94-0032

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: January 1994 Monthly Operating Report (MOR)

Enclosed is the January 1994 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,

for W. G. Gates
Vice President

WGG/mah

220170

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
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INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

JANUARY 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

The station operated at 100% power from January 1 through January 7 when power was decreased to 60% to repair a leaking tube in condenser FW-1A. One condenser tube was plugged and two previously plugged tubes were replugged.

Following repairs, a power increase to 100% commenced on January 8, but was stopped at 90% on January 9 due to a failed lower motor bearing on heater drain pump FW-5B and a subsequent motor winding failure on heater drain pump FW-5C. A power decrease was initiated on January 9 to 50% due to a failed level control valve on the heater drain tank.

After the failed level control valve was repaired, a power increase was initiated on January 10. The plant reached 90% power on January 11 and maintained that level. The heater drain pump FW-5C motor was replaced on January 12 and power was increased to 100% on January 13.

On January 18, a power excursion occurred (1508 MWth maximum) which resulted in reactor power exceeding 100% (1500 MWth) for greater than one hour. A Limiting Condition for Operation (LCO) was entered in accordance with Technical Specifications. Actions were taken to reduce reactor power and the LCO was exited. No Technical Specifications related to power level were violated. The cause of the power excursion was determined to be improper flushing of a Chemical and Volume Control System (CVCS) purification ion exchanger following the addition of new resin. FCS remained at 100% power through the end of January.

On January 30, Diesel Generator No. 1 was removed from service for scheduled maintenance and modification.

The following NRC inspections were completed during this reporting period:

IER No.	Description
94-01	Station Blackout Program
94-02	Erosion/Corrosion - Flow Accelerated Corrosion

94-06 Special Inspection: Auxiliary Feedwater Event (LER 93-019), Toxic Gas Monitor Event (LER 93-020), and Power Excursion Event

The following LERs were submitted during this reporting period:

LER No.	Description
93-018	Reactor Trip due to Turbine Trip on Low Electro-Hydraulic Fluid Pressure
93-019	Auxiliary Feedwater Pumps Inoperable due to Inappropriate Testing Lineup
93-020	Ventilation Mode Requirement not met while Toxic Gas Monitors were Inoperable

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of January, no PORV or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

RCS leak rate was steady throughout the month. The nominal leak rate was approximately 0.10 gpm. No degrading trends were noted; the reactor coolant system continues to have minimal leak rates.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
159	This amendment changed the Technical Specifications setpoint limit for the degraded-voltage protection system (referred to as the Offsite Power Low Signal).
160	This amendment implemented Generic Letters 86-10 and 88-12 by removing the fire protection requirements from the Technical Specifications and placing these requirements in the Updated Safety Analysis Report.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JANUARY 1994

- Replaced two circuit boards on the digital computer controller AI-208B-03.
- Installed a new phase meter on electrical control panel CB-20.

- Replaced a relay on the backup heater for Pressurizer Bank No. 3 Group No. 9 (MCC-4B-1-C02).

- Replaced 2 resistors and the circuit board on the preamplifier for excore detector NT-001.

- Replaced the mounting bolts on the raw water outlet valve operators from Component Cooling Water (CCW) Heat Exchangers AC-1A, AC-1C, and AC-1D.

- Replaced the housing, hand jack screw, O-ring and mounting bolts on the raw water outlet valve operator HCV-2881B-O from CCW Heat Exchanger AC-1B.

6. OPERATING DATA REPORT

   Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

   Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

   Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

   Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE FEBRUARY 07, 1994
COMPLETED BY M. A. HOWMAN
TELEPHONE 402-533-6939

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: JANUARY 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	744.0	178418.0
12. Number of Hours Reactor was Critical	744.0	744.0	138435.7
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	744.0	136807.2
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1071612.6	1071612.6	180357412.2
17. Gross Elec. Energy Generated (MWH)..	363016.0	363016.0	59451040.2
18. Net Elec. Energy Generated (MWH)....	346505.5	346505.5	56719568.4
19. Unit Service Factor.....	100.0	100.0	76.7
20. Unit Availability Factor.....	100.0	100.0	76.7
21. Unit Capacity Factor (using MDC Net)	97.4	97.4	68.9
22. Unit Capacity Factor (using DER Net)	97.4	97.4	67.2
23. Unit Forced Outage Rate.....	.0	.0	4.2

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	FEBRUARY 07, 1994
COMPLETED BY	M. A. HOWMAN
TELEPHONE	402-533-6939

MONTH JANUARY 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	488	17	487
2	488	18	487
3	488	19	487
4	488	20	487
5	489	21	487
6	488	22	488
7	474	23	487
8	292		487
9	396	25	487
10	248	26	487
11	422	27	487
12	428	28	487
13	483	29	487
14	487	30	486
15	486	31	486
16	487		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending January 31, 1994

1. Scheduled date for next refueling shutdown. — March 11, 1995

2. Scheduled date for restart following refueling. — April 29, 1995

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — No

   a. If answer is yes, what, in general, will these be? — N/A

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — No

   c. If no such review has taken place, when is it scheduled? — Prior to April 1995

4. Scheduled date(s) for submitting proposed licensing action and support information. — No submittal planned

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — **

6. The number of fuel assemblies:
   a) the core — 133 Assemblies
   b) the spent pool — 570 Assemblies
   c) spent fuel pool storage capacity — 729 Assemblies
   d) planned spent fuel pool storage capacity — 1083 Assemblies

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995 Outage*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed in 1994.

** OPPD is planning to utilize CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16. Additionally, if NRC approval of the CENTS code is obtained by May 1994, it will also be employed.

Prepared by [signature] Date 2-15-94

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE February 8, 1994
COMPLETED BY M. A. Howman
TELEPHONE (402) 533-6939

REPORT MONTH January 1994

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
94-01	01/07/94	F	0	A	4		HC	HCEXCH	On January 7, power was decreased to 66% to repair a leaking tube in condenser FW-1A. One condenser tube was plugged and two previously plugged tubes were replugged.
94-02	01/09/94	F	0	A	4		HH	VALVES	Following the above repair, a power increase commenced on January 8. On January 9, at 90% power, a level control valve on the heater drain tank failed, resulting in the initiation of a power reduction to 50%. After repair of the failed level control valve, a power increase to 90% commenced on January 10. The plant reached 90% power on January 11 and maintained that level. The heater drain pump FW-5C motor was replaced on January 12 and power was increased to 100% on January 13.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Te
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)