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February 13, 1994 C311-94-2020

U. S. Nuclear Regulatory Commission Attn: Document Control Lesk Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)

Operating License No. DPR-50

Docket No. 50-289

Monthly Operating Report for January 1994

Enclosed are two copies of the January 1994 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton

Jeg Broughton

Vice President and Director, TMI

WGH

Attachments cc: Administrator, Region I TMI Senior Resident Inspector

OPERATIONS SUMMARY January 1994

The plant entered and ended the month operating at approximately 100% power. The plant electrical output averaged approximately 816 MWe during the period.

MAJOR SAFETY RELATED MAINTENANCE

During January the following major safety related maintenance was performed:

Makeup Pump MU-P-1B

Makeup Pump MJ-P-1B was removed from service due to high vibration on the outboard bearings and an outboard mechanical seal leak. After disassembly of the bearing housing, inspection of the outboard thrust bearing identified no abnormal conditions while inspection of the outboard line bearing found excessive wear. The outboard line bearing was replaced. During reassembly, a new outboard mechanical seal was also installed. MU-P-1B test run and although vibration levels were found acceptable, the outboard seal had excessive leakage (pencil stream). In accordance with a Plant Engineering recommendation, the torque on the seal gland plate was relaxed and the leakage was reduced to "dripping". At present, the leak-off is being monitored.

Boric Acid Recycle Pump WDL-P-13A

Boric Acid Recycle Pump WDL-P-13A was removed from service to repair a mechanical seal leak. The pump was disassembled and a new seal installed. After pump reassembly, the pump was tested satisfactorily and was returned to service.

Intermediate Closed Cooling Water Pump IC-P-1B

Intermediate Closed Cooling Water Pump IC-P-1B was removed from service due to a noisy outboard motor bearing. After the bearing was replaced, IC-P-1B motor testing was completed satisfactorily and the pump returned to service.

OPERATING DATA REPORT

COMPLETED BY TELEPHONE 1 NOTES: 8 9 4 6 T, GIVE REASON	(717) 9	48-8191
8 1 9 4 6 EASON		
8 1 9 4 6 EASON		
):		
	YR-TO-DATE	
744.0		
744.0	744.0	93970.2
0.0	0.0	2283.8
744.0	744.0	92848.7
0.0	0.0	0.0
1908743		
642841		
606912	606912	71671431
	100.0	54.5
100.0		
	103.8	53.6
103.8	99.6	51.4
103.8	0.0	
103.8 99.6 0.0		
	103.8 99.6	100.0 100.0 100.0 100.0 103.8 103.8 99.6 99.6 0.0 0.0

AVERAGE DAILY UNIT P'WER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE February 13, 1994
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: JANUARY

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGL DALLY POWER LEVEL (MWe-NET)
1	819	17	812
2	819	18	810
	821	19	797
4	819	20	800
5	818	21	797
6	821	22	810
	820	23	814
	821	2.4	816
9	820	25	815
1.0	815	26	821
11	811	27	820
12	816	2.8	817
1.3	819	29	818
1.4	822		820
15	822	31	822
16	817		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January 1994

DOCKET NO. 50-289 UNIT NAME TMI-1 DATE February 13, 1994 COMPLETED BY W. G. Heysek TELEPHONE (717) 948-8191

No.	Diets	Type ¹	Duration (House)	Rosson	Method of Shating Down Rawson	Liourstee Event Report#	System Code * & *	Coreposerst Code	Cause & Corrective Active to Prevent Recorrence
						None			

F Forced S Scheduled

Rouson

A-Espaperers Failure (Explain) B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & Licensing Examination.

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method

1-Marsari

2-Manual Scram

3-Automatic Scram

4-Other (Explain)

Existing G - Instructions for restriction of Pata Entry Sheets for Lineus, invent Report (LER)

File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

- 1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
- 2. Scheduled date for next refueling shutdown: September 8, 1995
- 3. Scheduled date for restart following current refueling: NA
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? NO
- Scheduled date(s) for submitting proposed licensing action and supporting information: NA
- 6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
 - a) GPUN will continue to improve the review process by submittal of Technical Specification Change Requests (TSCR) concerning the reloadrelated areas of fuel assembly reconstitution and removal of cycledependent core limits to the Core Operating Limits Report (COLR).
 - 1) The B&W Owners Group Topical Report BAW-2149, "Evaluation of Replacement Rods in BWFC Fuel Assemblies", December 1991, was approved in April 1993. This report justifies the use of up to ten replacement stainless steel rods located anywhere in a single fuel assembly based on currently-approved methodology. BAW-2149 provides the basis for reconstitution repairs of BWFC Mark B assemblies to be done under the provisions of 10 CFR 50.59 (i.e., the repair does not represent an unreviewed safety question). TSCR 232 was submitted in August 1993 in response to Generic Letter 90-02, Supplement 1 referencing BAW-2149.
 - TSCR 234 was submitted in November 1993 consistent with GL 88-16 and the BAW-10179P "Safety Criteria and Methodology for Acceptable Cycle Reload Analyses" Safety Evaluation (March 16, 1993) for the removal of cycle-specific protective and maximum allowable setpoint limits for axial power imbalance and other applicable cycle-dependent limits.
- The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 601
- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase 1 of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.