

**ENCLOSURE
NOTICE OF VIOLATION**

Northeast Nuclear Energy Company
Millstone Nuclear Power Station
Units 1 and 2

Docket No.: 50-245; 50-336
License No.: DPR-21; DPR-65

During an NRC review, of previous inspections, three violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, the violations are listed below:

- A. Unit 1 Technical Specification 6.8.1 requires procedures to be established and implemented. Procedure SP412C, "Reactor Low-Low Water Level Functional Test/Calibration," requires, in part, that one train of the isolation condenser initiation logic be reset before testing the other train. Specifically, Step 6.2.15 requires the operator to place the control switch for valves IC-6 and IC-7 to "close" prior to resetting the logic and verify the logic relay reenergizes.

Contrary to the above, on September 2, 1992 the operator did not place the operating switches for IC-6 and IC-7 to "close," and did not verify that the logic relay was reenergized. Therefore, the 'A' train logic was not reset prior to testing the other train causing an inadvertent actuation of the isolation condenser.

This is a Severity Level IV Violation (Supplement I).

- B. Unit 1 Technical Specification 4.13 requires, in part, that inservice testing of ASME Boiler and Pressure Vessel Code equivalent valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code. ASME Boiler and Pressure Vessel Code Section XI, Article IWA 5262 requires, in part, that gauges be calibrated before their use for inservice tests.

Contrary to the above, as of September 20, 1991, diesel air start system pressure gauges PI-33-7 and PI-33-10 were not calibrated before use for an inservice test conducted in accordance with procedure SP668.12, "Diesel Air Start Check Valve Operability Test."

This is a Severity Level V Violation (Supplement I).

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- C. Unit 2 Technical Specification 4.6.1.1.a requires, in part, that valve 2-MS-255 (upstream vent for No. 1 steam generator MSIV bypass valve) be locked shut.

Contrary to the above, from August 27 to September 10, 1991, valve 2-MS-255 was shut but not locked shut.

This is a Severity Level IV Violation (Supplement I).

No response is required.

Dated at King of Prussia
this 3rd day of February, 1994