

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

February 15, 1994

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT HOPE CREEK GENERATION STATION UNIT 1 DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for January are being forwarded to you with the summary of changes, tests, and experiments that were implemented during January 1994 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours, games Clancy for RSHOV

R. J. Hovey General Manager -Hope Creek Operations

DR:WS:JC Attachments

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The Energy People

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OPERATING DATA REPORT

DOCKE	T NO. 50-154
UNIT	Hope Creek
DATE	02/09/94
COMPLETED BY	V. Zabielski
TELEPHONE	(609) 339-3506

This Vr To

OPERATING STATUS

1. Reporting Period January 1994 Gross Hours in Report Period 744

2.	Currently	Authorize	d Power	Level	(MWt)	3293
	Max. Depen	d. Capaci	ty (MWe	-Net)		1031
	Design Ele	ctrical R	ating ()	MWe-Net	:)	1067

3. Power Level to which restricted (if any) (MWe-Net) None

20 TROPPOLITE TOT TOPPOLIT . TT PILL	4. Reasons f	or	restri	lction	(if	any
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5.	No. of hours reactor was critical	Month 744.0	Date 744.0	Cumulative 53567.0
6.	Reactor reserve shutdown hours	0.0	0.0	0.0
7.	Hours generator on line	744.0	744.0	52776.5
8.	Unit reserve shutdown hours	0.0	0,0	0.0
9.	Gross thermal energy generated (MWH)	2441877	2441877	168405247
10.	Gross electrical energy generated (MWH)	824100	824100	55788054
11.	Net electrical energy generated (MWH)	<u>792184</u>	792184	53319868
12.	Reactor service factor	100.0	100.0	85.8
13.	Reactor availability factor	100.0	100.0	85.8
14.	Unit service factor	100.0	100.0	84.6
15.	Unit availability factor	100.0	100.0	84.6
16.	Unit capacity factor (using MDC)	103.3	103.3	82.9
17.	Unit capacity factor (Using Design MWe)	99.8	<u>99.8</u>	80.1
18.	Unit forced outage rate	0.0	0.0	4.4
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 Shutdowns scheduled over next 6 months (type, date, & duration): None
 If shutdown at end of report period, estimated date of start-up:

N/A

OPERATING DATA REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

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COMPLETED BY	V. Zabielski
TELEPHONE	(609) 339-3506

MONTH January 1994

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
						None

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-354
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DATE	02/09/94
COMPLETED BY	V. Zabielski
TELEPHONE	(609) 339-3506

MONTH January 1994

DAY A	VERAGE DAILY POWER LEVEL (MWe-Net)	DAY AVE	RAGE DAILY POWER LEVEL (MWe-Net)
1.	1068	17.	1066
2.	1058	18.	1076
3.	1068	19.	1076
4.	1067	20.	1062
5.	1066	21.	1072
6.	1077	22.	1070
7.	1068	23.	1057
8.	1057	24.	1052
9.	1068	25.	1075
10.	1068	26.	1067
11.	1067	27.	1074
12.	1071	28.	1061
13.	1069	29.	1016
14.	1057	30.	1069
15.	1079	31.	1057
16.	1058		

REFUELING INFORMATION

DOCKET NO. <u>50-354</u> UNIT <u>Hope Creek 1</u> DATE <u>February 12, 1994</u> COMPLETED BY <u>S. Hollingsworth</u> TELEPHONE (609) 339-1051

MONTH January 1994

1.	Refueling	information	has	changed	from	last	month:

Yes No X

2. Scheduled date for next refueling: 3/5/94

3. Scheduled date for restart following refueling: 4/23/94

4. A. Will Technical Specification changes or other license amendments be required?

Yes No X

B. Has the Safety Evaluation covering the COLR been reviewed by the Station Operating Review Committee?

Yes No X

If no, when is it scheduled? 2/23/94

- Scheduled date(s) for submitting proposed licensing action: Not scheduled yet.
- 6. Important licensing considerations associated with refueling: $\underline{N/A}$
- 7. Number of Fuel Assemblies:

8.

9

 A. Incore B. In Spent Fuel Storage (prior to refueling) C. In Spent Fuel Storage (after refueling) 	764 1008 1240
Present licensed spent fuel storage capacity: Future spent fuel storage capacity:	<u>4006</u> 4006
Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: (Does allow for full-core offload) (Assumes 244 bundle reloads every 18 months until (Does not allow for smaller reloads due to improv	<u>5/3/2006</u> (EOC13) then) red_fuel)

HOPE CREEK GENERATING STATION

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MONTHLY OPERATING SUMMARY

January 1994

Hope Creek entered the month of January at approximately 100% power. The unit operated at full power through the end of the month without any major power reductions or plant trips. As of January 31, 1994 the plant has been on line for 56 consecutive days.

Summary of Safety Evaluation

Design Change Packages

4HE-0014

This modification includes installation of a condensate diversion baffle, a two inch drain connection, and a trapped condensate drain line on the suction line to the Radwaste Crystallizer Vapor Compressor. Additionally, it modifies the existing casing condensate drain line with an isolable steam trap that allows continuous automatic removal of condensate.

The added condensate drip leg and steam trap are non-safety related items. They do not directly or indirectly alter the design or affect the operating procedures for the system.

A leak or failure of one of the proposed condensate drip lines would have far less radiological impact then the worst case release scenario from a liquid, solid, or gaseous waste system. The fluid that would be released has already been decontaminated as a result of being evaporated in the crystalizer.

Therefore, this DCP does not increase the probability or consequences of an accident previously described in the SAR and does not involve any Unreviewed Safety Question.

This DCP involves the installation of a minimum flow bypass line for the Liquid Radwaste Concentrator Feed Pumps for their protection from operating under dead head conditions. The activities of this modification will not alter the design, material, or construction standards applicable to the evaporator equipment. Any spill which may occur would be contained within the plant Floor Drain Collection System and not released directly from the plant.

Therefore, this DCP does not increase the probability or consequences of an accident previously described in the SAR and does not involve any Unreviewed Safety Question.

This modification installs integral flow regulator and controller to the seal water lines of the following pumps;

- A & B Waste Evaporator Concentrate Waste Transfer Pumps,
- A & B Waste Evaporator Recycle Pumps,
- A & B Concentrate Waste Pumps,
- A & B Waste Neutralizer Pumps,

and the Decontamination Concentrate Waste Pump.

4EC-3348 Package 12

4EC-3348 Package 15 The Flow indicators/controllers are designed to fail open. A random failure of the seal water line will result in leakage to the radwaste system which can be manually isolated in the pump room after detection.

Therefore, this DCP does not increase the probability or consequences of an accident previously described in the SAR and does not involve any Unreviewed Safety Question.

Summary of Safety Evaluation

Temporary Modifications

94-001, & 002

A temporary Jumper was installed around "A" and "C" Traveling Screen Flow Switch. The jumper simulates a flow signal which allows the travelling screens to operate. The flow sensor is inoperable and a replacement is on order. Equipment malfunctions postulated by the installation of this temporary modification are bounded by the single active failure criteria outlined in the SAR.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.