

NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-72

FLORIDA POWER CORPORATION, ET AL

CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT

DOCKET NO. 50-302

INTRODUCTION

By letter dated October 28, 1981, Florida Power Corporation (FPC) submitted Technical Specification (TS) Change Request No. 71 for Crystal River Unit No. 3 (CR-3) to (1) clarify TS 3.3.3.6 with regard to specifying which post-accident monitoring instrumentation readouts are supplemented with recorders, (2) decrease from two to one the "minimum channels operable" requirement for the Power Range Nuclear Flux instrumentation in TS Table 3.3-10, and (3) specify the measurement range of Reactor Coolant Total Flow from percent to 1b/hr to reflect actual indication.

EVALUATION

TS 3.3.3.6 presently requires post-accident monitoring instrumentation channels to be operable with readouts and recorders in the control room. Pursuant to the requirements of NUREG-0578, additional post-accident monitoring channels were installed at CR-3 with only readouts in the control room. TS Amendment No. 38 to TS Table 3.3-10 added this additional instrumentation but did not identify the fact that this new instrumentation contained only readouts and no recorders.

This proposed change to TS 3.3.3.6 will clarify which post-accident monitoring instruments require recorders in the control room.

CR-3 TS Table 3.3-10 presently requires a minimum of two Power Range Nuclear Flux channels to be operable. Babcock and Wilcox (B&W) Standard Technical Specifications (STS), Rev. 4, requires a minimum of one Power Range Nuclear Flux channel to be operable. The B&W STS, Rev. 4, also contain a "Required No. of Channels" column with an associated action statement in STS 3.3.3.6 if the number of channels operable decreases below the number specified. For Power Range Nuclear Flux channels, this "Required No. of Channels" number is two. The present CR-3 TS was formatted on B&W STS, Rev. 3, which does not contain a "Required No. of Channels" column nor an action statement in TS 3.3.3.6 if only one Power Range Nuclear Flux channel was operable.

This proposed change would have made the "Minimum Channels Operable" column of TS Table 3.3-10 inconsistent with the B&W STS, Rev. 4, due to the absence of a "Required No. of Channels" column and an associated action required for this condition. The intent of B&W STS to have two Power Range Nuclear Flux channels operable or be in an action statement is not met. Therefore, this portion of the proposed TS changes is considered unacceptable.

TS Table 3.3-10 indicates the measurement range of the post-accident monitoring instrumentation. The measurement range for Total Reactor Coolant Flow is listed as 0-110% full flow. The actual installed measurement range is $0-160 \times 10^6$ lb/hr.

The proposed change to TS Table 3.3-10 would replace the measurement range for Total Reactor Coolant Flow to 0-160 x 10^6 lb/hr which is consistent with the installed instruments range.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR \$51.5(d)(4), that an environmental impact statement, congative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

Based on the considerations discussed above, we have concluded, except for the item decreasing the minimum channels operable for the power range nuclear flux instrument which is not authorized, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposition of the public will be conducted in compliance with the we ission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 23, 1982

The following NRC personnel have contributed to this Safety Evaluation: Barry Smith, Tom Stetka, John Rogge.