COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-11 Amendment No. 9

- 1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company dated August 19, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I:
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations: set forth in 10 CFR Chapter I:
 - The issuance of this amendment will not be inimical to the cormon defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

Technical Specifications (2)

The Technical Specifications contained in Appendix A. as revised through Amendment No. 9, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer, Chief Licensing Branch No. 2 Division of Licensing

Date of Issuane: November 18, 1982

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FACILITY OPERATING LICENSE NO. NPF-11 DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

 REMOVE
 INSERT

 3/4 4-19
 3/4 4-19

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Table 4.4.6.1.3-1
Reactor Vessel Material Surveillance Program Withdrawal Schedule

Specimen holder	Vessel location	Lead factor	Withdrawal time (Service Years)
117C4936G010	300°	0.6	10
117C4936G011	120°	0.6	30
117C4936G012	30°	0.6	Spare
Neutron Dosimeter	30°		1st Refuel Outage