03 APR 1986

MEMORANDUM FOR: Robert M. Bernero, Director
Division of Licensing, NRR

FROM: Richard W. Starostecki, Director
Division of Reactor Projects, Region I

SUBJECT: COMPLETION STATUS OF NINE MILE POINT UNIT 2 (DN 50-410)

This memorandum forwards Region I's 90-day report on the status of construction completion and readiness for operation of Nine Mile Point Unit 2. This report provides, in the enclosures, specific information regarding construction status, preoperational testing status, and Region I inspection program status including preoperational testing inspections, outstanding inspection items, and unresolved allegations.

Enclosure 1 presents the estimated construction status for Nine Mile Point Unit 2. On January 21, 1986, Niagara Mohawk Power Corporation revised the fuel load schedule to the week of May 5, 1986, a delay of approximately ten weeks. Although I believe this revised schedule is optimistic and based on recent performance will not be met, we will allocate the inspection resources to conduct the required inspections of activities, as they are completed. The licensee has also submitted to you a preliminary listing of nine systems which may not be completed prior to May 5, 1986, and for which preoperational testing deferrals may occur. I do not endorse the concept of deferring so many tests beyond fuel load. In fact, I would recommend allowing limited test exceptions once all of the pre-op tests have been done. Enclosure 2 presents the preoperational testing status including the list of the proposed deferrals.

The Region I Outstanding Items list for Nine Mile Point Unit 2 is presented in Enclosure 3. This list is all inclusive up to the date of issuance of this memorandum. We are in the process of determining the priority of each item as it relates to license issuance, fuel loading, initial criticality, and low power operation. This prioritization will be provided in a future memorandum.

We will evaluate the status of the plant approximately every 30 days and will notify you of any major impediments as they occur. Prior to a fuel load license decision we will provide NRR with a written assessment of the licensee's readiness for a license.

Original Signed By:

Richard W. Starostecki, Director
Division of Reactor Projects

Enclosures: as stated

cc: w/enclosures
S. Ebneter, DRS
T. Martin, DRSS

JE/14

OFFICIAL RECORD COPY 9MILE 2 90 DAY - 0001.0.0
04/02/86

9102060300 901011
PDR FOIA
PERSON 90-207 PDR

8/6-21

bcc (w/encls):
R. Gramm, DRP
S. Hudson, DRP
J. Linville, DRP

RI:DRP
Doerflein/cop/bc
4/2/86

RI:DRP
Linville
4/2/86

RI:DRP
Starostecki
4/2/86

OFFICIAL RECORD COPY

ENCLOSURE 1

CONSTRUCTION STATUS

The licensee estimates the Nine Mile Point Unit 2 construction to be 95% complete. The following summarizes those major work activities not yet completed:

Activity	% Complete
System Turnover	94 of 108 systems (87%) have been turned over to NMPC for preoperational testing
Building Turnover	47%
ASME N-5 Certification	68%
Large Bore As-Built	84%
Pre-Service Inspection (PT, MT, VT)	86%
Cat I Cable Pulls	96%
Cat I Terminations	98%
Penetration Sealing	31%
Structural Load Verification	98%
Equipment Qualification	96%
Open SWEC Documents (ACN, E&DCR, N&D)	32%

ENCLOSURE 2

PREOPERATIONAL TESTING STATUS
(MARCH 3, 1986)

A. Preoperational Test (POT)/Acceptance Test (AT) Status Summary

	(POTS)	(ATS)	Total
Total tests:	107	30	137
Procedures approved by JTG for testing:	99	27	126
Tests in progress:	51	23	74
Tests Completed:	34	21	55
Test Results Approved (JTG):	14	11	25
Test Results Approved (SORC):	3	0	3

B. Preoperational Test Deferrals

In a letter to NRR dated January 31, 1986, the licensee provided a listing of those systems for which preoperational testing, and post test reviews may not be completed prior to the scheduled fuel load date of May 11, 1986. The following is a list of those systems for which deferrals may be requested:

1. Feedwater Heaters and Extraction Steam (POT 8)
2. Turbine EHC Oil and Control (POT 23-1, 23-2)
3. Automatic Depressurization (POT 34)
4. Solid Radwaste (POT 41)
5. Off Gas (POT 42)
6. DBA Recombiner (POT 62)
7. Containment Leakage Monitoring (POT 81)
8. Containment Atmosphere Monitoring (POT 82)
9. Nitrogen System/Containment Inerting (POT 88-1, 88-2)

C. Status of Preoperational/Acceptance Tests

The following list indicates the completion status of preoperational/acceptance tests.

Test	System	% Test Complete	Results Approved	NRC* Insp.
POT-1	Main & Auxiliary Steam			
AT-2	Moisture Separators/RHTRS	100		
POT-3	Condensate	100		P/
POT-4	Condensate Storage & Transfer	100	JTG	P/
POT-5	Condensate Demin & Resin Regen.	100		P/
POT-6	Feedwater	5		
POT-7	Feedwater Control			
POT-8	Feedwater Heaters			P/
POT-9	Condenser Air Removal	100	JTG	P/
POT-10A	Circulating Water	5		
AT-10B	Acid Treatment	10		
AT-10C	Hypochlorite	10	JTG	
POT-11	Service Water			
AT-12	Traveling Water Screens	100	JTG/	
POT-13	Reactor Closed Loop Cooling	25		
POT-14	Turbine Closed Loop Cooling	80		
AT-15	Makeup Water Treatment	100	JTG/	
POT-16	Makeup Water Storage and Transfer	100	JTG/	
POT-17-1	Turbine Plant Sampling	97		
POT-17-2	Reactor Plant Sampling	5		
POT-17-3	Radwaste Plant Sampling	100		
POT-17-4	Post Accident Sampling			
POT-19-1	Instrument and Service Air	100		
POT-19-2	Loss of Plant Air - N2			
AT-20	Breathing Air			
AT-22A-1	Generator Seal Oil	100		
AT-22A-2	Main Lube Oil	100		
AT-22B	Turbine Lube Oil	100	JTG/	
POT-23-1	Turbine EHC -Electronic			
POT-23-2	Turbine EHC - Hydrualic	100	JTG/	
AT-24	Generator Ioslated Phase Bus Duct Cooling	100	JTG/	
AT-25	Clean Steam Reboiler & Aux. Condensate	100		
AT-26	Turbine Generator Stator Cooling Water	100	JTG/	
AT-27	Generator H2 & CO2 Gas			

Test	System	% Test Complete	Results Approved	NRC* Insp.
POT-28	Nuclear Boiler Instrumentation			
POT-29-1	Reactor Recirc.			
POT-29-2	Reactor Recirc. Flow Control			
POT-30	Control Rod Drive Hydraulics	5		
POT-31	Residual Heat Removal			
POT-32	Low Pressure Core Spray	100		P/W/
POT-33	High Pressure Core Spray	100		P/W/
POT-34	Automatic Depressurization			
POT-35	Reactor Core Isolation Cooling			P/
POT-36-1	Standby Liquid Control	95		P/
POT-36-2	Neutron Absorber			
POT-37	Reactor Water Cleanup			
POT-38	Fuel Pool Cooling and Cleanup	100	JTG/	P/
POT-39	Fuel Handling and Reactor Service	100	JTG/	P/
POT-40-1	Liquid Radwaste			
POT-40-2	Radwaste Process Computer			
POT-41	Solid Radwaste			
POT-42	Off-Gas			
POT-43	Fire Water Protection			
POT-44	Fire Protection Foam			
POT-45	Fire Protection CO2			
POT-46	Fire Protection Halon			
POT-47	Smoke, Flame and Temperature Detection			
AT-48	Auxiliary Boiler	100	JTG/	
AT-49-1	Hot Water & Glycol Heating	100	JTG	P/
AT-49-2	Turbine Bldg Hot Water & Glycol	100	JTG/SORC	P/
AT-49-3	Radwaste Bldg Glycol Heating	100	JTG/SORC	
AT-49-4	Reactor Bldg. Hot Water & Glycol	100	JTG/SORC	
AT-50	Domestic Water			
POT-52	Reactor Bldg HVAC			
POT-53-1	Control Bldg HVAC			
POT-53-2	Control Bldg Chilled Water			

Test	System	% Test Complete	Results Approved	NRC* Insp.
POT-53-3	Control Room Pressure Test			
AT-54-1	Normal Switchgear Bldg Ventilation	100	JTG/	
AT-54-2	Lithium Bromide Chilled Water			
POT-55	Turbine Bldg. Ventilation			P/
POT-56-1	Radwaste Bldg Ventilation			
POT-56-2	Radwaste Bldg Pressure/ Flow Test			
POT-57	Diesel Generator Bldg Ventilation	100		
AT-58-1	Screenwell Diesel Firepump	100		
AT-58-2	Misc. Vent Service Bldg.	25		
AT-58-3	Auxiliary Boiler Ventilation	100	JTG	
POT-59-A	Electric Tunnels Ventilation			P
POT-60	Drywell Cooling			
POT-61-1	Primary Containment Purge			
POT-61-2	Standby Gas Treatment			
POT-62	DBA Hydrogen Recombiner			
POT-63-1	Reactor Bldg. Equipment Drains	100		
POT-63-2	Reactor Bldg. Floor Drains	100		
POT-64-1	Turbine Bldg. Equipment Drains	100		
POT-64-2	Turbine Bldg. Floor Drains	100		
POT-65	Radwaste Bldg. Drains	100		
AT-66-1	Reserve Transformer Area Drains	100	JTG/	
AT-66-2	Main Transformer Area Drains	5		
POT-66-3	Diesel Generator Bldg. Floor Drains	25		
AT-66-4	Screenwell Bldg. Drains	5		
AT-66-5	Service Bldg. Drains	100		
AT-66-6	Control Bldg. Drains	100		
AT-66-7	Auxiliary Boiler Bldg. Drains	100		

Test	System	% Test Complete	Results Approved	NRC* Insp.
POT-66-8	Condensate Storage Tank Bldg. Drains	100		
POT-66-9	Main Stack Drains	100		
POT-66-10	Reactor Bldg. Mat Drains	Deleted		
POT-67	Drywell Equipment and Floor Drains	100		
POT-71	Uninterruptable Power Supplies	100	JTG/	
AT-73-1	125V Normal DC Distribution	100	JTG/	
POT-73-2	24/48V DC Distribution	100	JTG/	
POT-74-1	125V Emergency DC Distribution Div. 1	100	JTG/	
POT-74-2	125V Emergency DC Distribution Div. 2	100		W
POT-74-3	Division III Emergency DC	100		P/W
POT-75	Station Emergency Lighting			
POT-76	Communications			
POT-78	Remote Shutdown			
POT-80A-1	Digital Radiation Monitor			
POT-80A-2	Gaseous Effluent Monitor			
POT-80B	Main Steam Line Monitoring			
POT-81	Containment Leakage Monitoring			
POT-82	Containment Atmosphere Monitoring			
POT-83	Primary Containment Isolation			
POT-84	Reactor Bldg. Polar Crane	100	JTG/	P/
POT-85	Reactor Coolant Leak Detection			
POT-86	Loose Parts & Vibration Monitoring			
POT-88-1	Containment Inerting			
POT-88-2	Contaminant Inerting			
POT-90	Seismic Monitoring	30		
POT-91	Process Computer	100		
POT-92-1	Source Range Monitoring			
POT-92-2	Intermediate Range Monitoring			
POT-93	Rod Block Monitoring			

Test	System	% Test Complete	Results Approved	RC* Insp.
POT-94	Traverse Incore Probe			
POT-95A	Rodworth Minimizer			
POT-95B	Rod Sequence Control			
POT-96	Reactor Manual Control	100		
POT-97	Reactor Protection			P/
POT-100A-1	Division I Diesel Generator	5		/W
POT-100A-2	Division II Diesel Generator	5		/W
POT-100B	HPCS Diesel Generator	5		P/W
AT-101-1	Turbine Bldg. Crane			
AT-101-2	Radwaste Bldg. Crane			
AT-104	Security			
POT-106	Redundant Reactivity Control			
POT-200	Secondary Cont. Leak Rate			
POT-201	Structural Integrity & ILRT			
POT-300	Loss of Offsite Power/ ECCS Functional Test			P/

*P=procedure review
W=witness test
R=results review

ENCLOSURE 3

NRC REGION I INSPECTION STATUS

A. Preoperational Testing and Operational Readiness

The following list shows the status of incomplete inspection activities for Nine Mile Point Unit 2. The current percent completion status of each activity is indicated. For the majority of the areas, this inspection status is consistent with the licensee's testing schedule or the state of completion of the program area. The Region I staff is evaluating resource needs to complete these inspection activities prior to fuel load.

Area	Inspection % Complete
Preoperational Quality Assurance	80%
Overall preoperational test program	30%
Preoperational test program implementation	5%
As-built verification	25%
Preoperational Test procedure review	
Mandatory*	60%
Primal	50%
Preoperational test witnessing	
Mandatory	30%
Primal	25%
Test results review	
Mandatory	0%
Primal	0%
Operations staffing and procedures	5%
Technical Specification Review	75%
Operations Quality Assurance	5%
Maintenance	5%
Fire Protection	50%
Fuel Receipt	100%
Surveillance	10%

Radiological Controls	35%
Radwaste	20%
Security	40%
Emergency Planning	5%

*Consists of Engineered Safety Features preoperational tests, Loss of Offsite Power test, Reactor Protection System preoperational test, Integrated Leak Rate Test, and Reactor Pressure Vessel hydrostatic test.

B. Outstanding Items

The following lists identify, by category, all outstanding items for Nine Mile Point Unit 2. Region I is prioritizing these items to ensure all those required for fuel load are completed prior to license issuance.

IE BULLETINS (10)

74-BU-13	Improper factory wiring of GE Motor Control Centers
79-BU-01	Environmental Qualification of ASCO Solenoid Valves
79-BU-02	Pipe support baseplate design
79-BU-14	Seismic as-built piping analysis
79-BU-16	Access to vital areas
79-BU-28	Malfunction of NAMCO Limit Switches
80-BU-07	BWR jet pump assembly failure
80-BU-16	Misapplication of Rosemount pressure transmitters
84-BU-02	Failure of GE HFA relays
84-BU-03	Refueling cavity water seal

CONSTRUCTION DEFICIENCIES (53)

80-00-03	Spent fuel cooling HX test data
82-00-10	GE HFA relays incorrect setting
82-00-12	Clow valves not heat treated
82-00-17	Defective ITT Barton pressure transducers
83-00-03	Square D switches failed environment qualification
83-00-04	Separation criteria for PGCC panels
83-00-06	Accident analysis source terms
83-00-07	ITE Gould circuit breaker failed EQ
83-00-10	Uncertified inspected accepted installation
83-00-12	Uncertified inspectors performed work
83-00-14	Failure of Power Conversion battery chargers
83-00-17	Hydrogen Recombiner microswitch failed EQ
83-00-18	ECCS pump operation vortex formation
83-00-19	MSIV actuator failed EQ
83-00-23	Design of reactor building roof
84-00-02	Undersized Cives shop welds
84-00-06	Excessive wear on Pacific Air products Linear Converters
84-00-09	Hydraulic transient for pump restart
84-00-14	PGCC separation requirements
84-00-18	Improper terminations in HPCS DG control panel
84-00-21	Agastat relays
84-00-26	Corrosion on MSIV spool seats
84-00-28	Inspection of pipe flange connections
84-00-32	HPCS DG control panel wiring separation
84-00-33	RCIC steam line vents
84-00-34	Erratic operation of Rosemount trip units

84-00-36 Fire dampers failed to close
84-00-40 Seismic analysis of containment purge isolation valves
84-00-42 Closure time of isolation valves
84-00-43 Guyon sock-o-let material certification
84-00-46 Clow valves miswired
84-00-48 Separation criteria for cables
84-00-49 Minimum wall violations of pipe welds
84-00-51 Traceability of spare parts
84-00-53 RCI welds undersized
84-00-55 Defective Topaz Inverters
84-00-56 Overpressurization of LPCS piping
85-00-01 HPCS control panel wiring discrepancies
85-00-02 Improperly cured containment zinc primer
85-00-04 MSIV latching bearing failure
85-00-08 DG jumpers prevent LOCA start
85-00-14 Undersized DG load shedding timers
85-00-15 Wiring missing for LPCS valve
85-00-18 Interstate tube steel displayed linear indications
85-00-19 QA involvement in valve disassembly
85-00-21 Setpoint of RCIC drain tap
85-00-22 Limitorque valve operators not seismically qualified
85-00-24 Rejectable solder connections
85-00-25 MSIV body heat treatment
85-00-27 Failure of DG to start in emergency mode
85-00-29 Undersized resistor in DG exciter circuitry
85-00-30 Design interference around framing steel
86-00-01 Increased Secondary Containment drawdown time

VIOLATIONS (21)

82-11-09 Use of uncertified inspectors
83-06-03 Inadequate review of FSAR information
83-18-70 Inadequate RT interpretations
83-18-71 Deficient electrical installation procedures
83-18-74 Inadequate PT weld examinations
83-18-75 Inspection records do not list design document
83-18-76 Design changes not incorporated
83-18-77 Design changes not at work location
83-18-79 Inadequate PQA coverage of vendors
83-18-83 Improper PT examinations
83-18-86 Inadequate corrective actions to RT problems
83-18-87 Lack of control over risk release design changes
83-18-92 Ineffective NMPC audit programs
83-18-95 Inaccurate RT reader sheets
84-06-08 Inadequate Primary Containment housekeeping
84-18-08 Inadequate equipment PM implementation
85-03-03 Protection of DG and associated components
85-36-01 Remote shutdown panel bolting hardware undersized

85-42-02 Minimum cable bend radius violations
85-42-03 Installation of NMS cables without approved procedures
86-01-XX Incomplete HPCS DG Preop. test acceptance criteria

UNRESOLVED AND FOLLOWUP ITEMS (106)

80-09-01 Control of component shelf life
83-01-08 Refuse in tube steel
83-01-09 PM program implementation
83-08-02 GE compliance to Reg Guide 1.100
83-08-04 MCC seismic qualification
83-08-05 Vendor inspection attributes for wiring
83-12-11 Sway strut design assumptions
83-15-01 DG PM program implementation
83-16-07 CRD Restraint beam assembly
83-18-A1 Conduct of electrical QC inspections
83-18-C1 Misuse of unsatisfactory IRs
83-18-D1 Control of deferred work inspection
83-18-F1 Ineffective inspection conduct
83-18-H1 Poor workmanship
83-18-I1 Inadequate vendor QC programs
83-18-J1 Nonconformance reports not correct
83-18-L1 Nonconforming conditions not properly identified
83-18-M1 Inadequate QA/QC program
83-18-N1 Document control program problems
83-18-Q1 Poor QA/QC management
83-18-R1 Inadequate SWEC PQA implementation
83-18-S1 Material traceability
83-18-T1 Evaluation of 50.55(e) conditions
83-18-17 Cable damage improperly dispositioned on N&D
83-18-41 Visual and PT examination of pipe welds
83-18-42 Deficient ITT shop and field weld RTs
83-18-43 Retention of RT film
83-18-46 Undersized RCI welds
83-18-48 Cable tray support welds undersized
83-18-49 Undersized Cives structural welds
83-18-96 Timeliness of QC inspection conduct
83-18-97 Improper QC inspection conduct
83-18-98 Lack of QC inspection attributes
83-18-99 QC records lack design document notation
84-09-04 Conduct OI investigation of employee harassment
84-10-02 Security management staffing
84-10-05 Pre Fuel load security program audit
84-10-06 Security records for Unit 2
84-10-07 Maintenance of security systems
84-10-10 Water drain and tunnel security survey
84-10-11 Vital area barrier analysis
84-10-12 Security system full load test
84-10-13 Completion of site lighting
84-10-14 Completion of low light level CCTV
84-10-15 Completion of Unit 2 access control program

84-10-18 Completion of protected area perimeter detection aids
84-10-19 Completion of door alarm switches and card readers
84-10-20 Completion of central alarm station
84-10-21 Completion of site communications system
84-10-23 Security contingency plan drill
84-10-24 Procedures for unusual security events
84-13-02 Response to QC inspector concerns
84-18-04 Drawing control
84-18-09 Backlog of Corrective Action Requests
85-03-02 Replacement of unqualified flow transmitters
85-04-02 Control of Licensing commitments
85-06-03 Fit-up acceptance criteria for welding
85-06-04 CRD restraint installation tolerances
85-10-03 Witness nitrogen inerting system pre-op
85-10-04 Adequacy of pre-op procedure review
85-13-03 Battery pre-op procedure concerns
85-13-04 MSIV leak rate testing
85-13-05 Lack of timely QC inspection
85-13-06 CRD installation control of M&TE
85-19-03 SU QA implementation CAT corrective actions
85-19-04 RPV hydrostatic test results review
85-20-01 Review liquid radwaste pre-op testing
85-20-02 Review solid radwaste pre-op testing
85-20-03 Review HVAC/Gaseous waste testing
85-20-07 Review personnel training for HP and radwaste
85-20-08 Resolution of IEB 80-10
85-20-09 Staffing of rad controls organization
85-25-01 Open audit item on GE excluded equipment list
85-25-02 As-built program for GE designed systems
85-25-03 Diesel Generator test requirements
85-25-04 Review RPV as-built data
85-27-01 Evaluation of construction fires
85-30-01 Control of flushing program
85-31-01 Adequacy of weld sampling program
85-32-01 Procedure training program not adequate
85-32-02 Review Bulletins and Circulars
85-32-03 Review fuel receipt plans
85-32-04 Review ARM program
85-32-05 Review GEMS testing
85-32-06 Review DRMS calibration
85-32-07 Establish procedures for radwaste handling
85-32-08 Install/calibrate Main Steam line monitors
85-32-09 Complete shield surveys
85-34-01 20 foot separation zone suppression
85-34-02 Structural steel fire proofing
85-34-03 Construction joint fire seals
85-34-04 Fire detector installation
85-34-05 Fire dampers in day tank ductwork
85-34-06 NFPA code deviations
85-34-07 Completion of emergency lighting systems

85-37-01 Training for fuel load operation staff
85-45-01 Fire barriers and detection in reactor building not fully operational
85-46-01 Acceptance of PSI data
85-99-01 Resolution of AISC bolting deficiencies
85-99-08 Effects of schedular pressures on inspection conduct
85-99-13 Long term corrective action program
85-99-16 Adequacy of design change document
85-99-18 FSAR verification process
86-01-XX Inconsistent DG jacket water heater setpoints
86-01-XX Flow capacity of Standby Liquid Control System vs 10 CFR 50.62
86-03-XX POT-300 isolation valves inconsistent with FSAR

TMI Action Plan Items

Due Prior to Fuel Load (43 Items)

I.A.1.1 Shift Technical Advisor (Items 1 and 3)
I.A.1.2 Shift Supervisor Responsibilities
I.A.1.3 Shift Manning (Items 1 and 2)
I.A.2.1 Upgrading of RO and SRO Training and Qualifications (Item 4)
I.A.2.3 Administration of Training Programs
I.B.1.2 Evaluation of Management Organization
I.C.1 Short Term Accident Analysis and Procedure Review (Item 1)
I.C.2 Shift and Relief Turnover Procedures
I.C.3 Shift Supervisor Responsibility
I.C.4 Control Room Access
I.C.5 Feedback of Operating Experience
I.C.6 Verify Correct Performance of Operating Activities
I.C.7 NSSS Vendor Review of Procedures (Item 1)
II.B.4 Training for Mitigating Core Damage (Item 1)
II.D.1 Relief and Safety Valve Test Requirements (Item 2)
II D.3 Valve Position Indication
II.E.4.1 Dedicated Hydrogen Penetrations (Items 2 and 3)
II.F.1 Accident Monitoring Instrumentation (Items 1, 2a, b, c, d, e, and f)
II.F.2 Instrumentation for detection of Inadequate Core Cooling (Item 4)
II.K.1 IE Bulletins and Measures to Mitigate SBLOCA's and Loss of Feedwater Accidents (Items 5, 10, 22 and 23)
II.K.3.13 HPCI and RCIC Initiation Levels
II.K.3.15 Isolation of HPCI and RCIC Modification
II.K.3.22 RCIC Suction (Items 22A and B)

II.K.3.24 Space Cooling for HPCI/RCIC Modifications
II.K.3.27 Common Reference Level
II.K.3.28 Qualifications of ADS Accumulators
III.A.1.2 Upgrade Emergency Support Activities (Items 1A and B)
III.D.3.3 Inplant Radiation Monitoring (Items 1 and 2)

Due Prior to Full Power License (24 Items)

I.C.1 Short Term Accident Analysis and Procedures Review (Items 2 and 3)
I.C.7 NSSS Vendor Review of Procedures (Item 2)
I.C.8 Pilot Monitoring of Selected Emergency Procedures
I.G.1 Training During Low Power Testing
II.B.1 Reactor Coolant System Vents (Items 2 and 3)
II.B.2 Plant Shielding
II.B.3 Post Accident Sampling (Items 3 and 4)
II.B.4 Training for Mitigating Core Damage (Item 2)
II.E.4.2 Containment Isolation Dependability (Items 1 through 7)
II.K.3.16 Challenges and Failures to Relief Valves
II.K.3.18 ADS Actuation
II.K.3.21 Restart of CSS and LPCI
II.K.3.25 Power on Pump Seals (Items 25A and B)
III.D.1.1 Primary Coolant Outside Containment

Other (2 Items)

I.D.2 Plant Safety Parameter Display Console (Items 2 and 3)

C. Unresolved Allegations

There are currently 7 open allegations pertaining to Nine Mile Point Unit 2. Region I does not believe that these allegations should be a factor in the licensing process. The open allegations are summarized below.

RI-84-A-75 A contractor employee made allegations of improper cable terminations involving problems with dissimilar bus bar material, improper paperwork, bypassing Quality Control (QC) holdpoints, and intimidation of a QC inspector. An inspection found some of the concerns were valid and the licensee implemented adequate corrective action. Region I Office of Investigation investigated the intimidation issue and found it unsubstantiated. A Department of Labor (DOL) hearing found in favor of the QC inspector. This allegation remains open pending Region I review of the final DOL decision.

RI-84-A-86 A licensee employee alleged harassment of Quality Assurance auditors because of negative audit findings. This allegation remains open pending completion of an investigation initiated by Region I Office of Investigation.

RI-84-A-104 A contractor employee made an allegation concerning unmarked termination hardware in that there was no guidance for electrical inspectors to ensure the proper material bolting hardware was used. In addition the alleger stated that his concerns were ignored by his supervision. An inspection found that the different material types of bolting hardware were distinguishable and that there was no technical concern. This allegation remains open pending completion of an Office of Investigation report on the issue of supervisors ignoring the concerns.

RI-85-A-65 An anonymous allegation was made concerning improper installation of Neutron Monitoring System cables, undue pressure on workers, and Quality Control inspectors not doing their job. The allegation was given to the licensee's Quality First Program (Q1P) for investigation and resolution. The licensee found the allegation unsubstantiated, however, a subsequent allegation by a contractor employee (RI-85A-100 below) was made that this particular Q1P investigation was inadequate. Region I conducted an inspection of the Neutron Monitoring System cable installation and concluded there were no technical concerns on the adequacy of installation although there was a procedure

violation regarding the cable installation method. Another inspection also examined the QIP for adequacy and found that the QIP resolution of safety related concerns was satisfactory. Closure of this allegation is pending completion of documentation.

RI-85-A-100 A contractor employee made an allegation that the Quality First Program (QIP) investigation of the Neutron Monitoring System (NMS) cable overtension problem was inadequate. As noted above in allegation RI-85-A-65, Region I conducted inspections of the NMS cable installation and QIP and found both satisfactory with the exception of the procedural violation. This allegation remains open pending completion of the investigation by Region I Office of Investigation on the potential wrongdoing issues associated with the allegation.

RI-85-A-111 A contractor employee made an allegation concerning improper completion of Reactor Controls, Inc. (RCI) surveillance reports by Stone and Webster personnel. A Region I inspection found that the reviews by Stone and Webster personnel were part of the document turnover process and that there were no technical concerns. Closure of this allegation is pending final documentation.

RI-85-A-112 An anonymous allegation was made concerning mistorqued bolts on the Diesel Generator (DG) fuel oil lines and the adequacy of Quality First Program (QIP). As noted above, the QIP was inspected and the resolution of safety related concerns was found satisfactory. The specific problem of mistorquing DG fuel oil line bolts is awaiting an inspection.