



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
February 22, 1991

Docket No. 50-338

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: NORTH ANNA UNIT 1 (NA-1) - CYCLE 9 CORE SURVEILLANCE REPORT

The NA-1 Technical Specification (TS) 6.9.1.7 requires a Core Surveillance Report to be provided to the NRC 60 days prior to cycle criticality unless a written waiver is granted by the NRC staff.

By letter dated January 18, 1991, you requested that the 60-day advanced submittal requirement be waived by the NRC for the following reason.

The delay in making the submittal was unanticipated by you and was due to your need to temporarily redirect technical resources preparing the report of activities associated with your Surry 1&2 facility. These activities included a priority safety analysis to support continued operation of Surry 2 with one stuck control rod and the need to redesign the Surry 1 core during a scheduled refueling outage due to the discovery of failed fuel. Your January 18, 1991 submittal is less than 60 days prior to the information becoming effective for NA-1 Cycle 9 criticality scheduled for March 12, 1991. Therefore, in accordance with the NA-1 TS 6.9.1.7 reporting provision, you requested in your January 18, 1991 letter that the 60-day advance submittal requirement be waived by the NRC.

Your NA-1 Cycle 9 reload core was analyzed in accordance with the methodology documented in your approved Topical Report VEP-FRD-42, Revision 1-A, "Reload Nuclear Design Methodology," using NRC-approved codes as referenced in the Topical Report. The information in the report was developed in accordance with your Topical Report VEP-NE-1-A, "Relaxed Power Distribution Control Methodology and Associated FQ Surveillance Technical Specifications." The results of the analyses indicated that no key analysis parameters would become more limiting during Cycle 9 operation than the values assumed in the currently applicable NA-1 safety analysis. Further, the analyses demonstrate that the current NA-1 TS are appropriate and require no additional changes. The

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Core Surveillance Report consists of the specific NA-1 Cycle 9 values for N(z) and the axial flux difference limits based upon the currently approved total peaking factor (Fq) limit of 2.19.

Therefore, based on the above, a waiver is hereby granted to the NA-1 TS 6.9.1.7 reporting requirements.

Sincerely,

(Original signed by)

Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects - 1/II
Office of Nuclear Reactor Regulation

cc: See next page

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Mr. W. L. Stewart
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:

Mr. William C. Porter, Jr.
County Administrator
Louisa County
P.O. Box 160
Louisa, Virginia 23093

C. M. G. Buttery, M.D., M.P.H.
Department of Health
109 Governor Street
Richmond, Virginia 23219

Michael W. Maupin, Esq.
Hunton and Williams
P. O. Box 1535
Richmond, Virginia 23212

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30323

Mr. W. T. Lough
Virginia Corporation Commission
Division of Energy Regulation
P. O. Box 1197
Richmond, Virginia 23209

Mr. G. E. Kane, Manager
North Anna Power Station
P.O. Box 402
Mineral, Virginia 23117

Old Dominion Electric Cooperative
c/o Executive Vice President
Innsbrook Corporate Center
4222 Cox Road, Suite 102
Glen Allen, Virginia 23060

Mr. J. P. O'Hanlon
Vice President - Nuclear Services
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. E. Wayne Harrell
Vice President - Nuclear Operations
Virginia Electric and Power Co.
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. Martin Bowling
Manager - Nuclear Licensing
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. Patrick A. O'Hare
Office of the Attorney General
Supreme Court Building
101 North 8th Street
Richmond, Virginia 23219

Senior Resident Inspector
North Anna Power Station
U.S. Nuclear Regulatory Commission
Route 2, Box 78
Mineral, Virginia 23117