

November 24, 1982

Judge Gary J. Edles, Chairman  
Atomic Safety & Licensing Appeal Board  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Judge John H. Buck  
Atomic Safety & Licensing Appeal Board  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Judge Reginald L. Gotchy  
Atomic Safety & Licensing Appeal Board  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

In the Matter of  
METROPOLITAN EDISON COMPANY, ET AL.  
(Three Mile Island Nuclear Station, Unit No. 1)  
Docket No. 50-289

Dear Appeal Board Members:

It has been brought to my attention that the figures referred to in the Affidavit of Brian W. Sheron and the statements of professional qualifications of Mr. Sheron and Mr. Jensen were not enclosed with the "NRC Staff Comments in Response to Appeal Board Memorandum and Order of November 5, 1982" filed on November 22nd as they should have been. Copies of the omitted items are attached. The Staff regrets any inconvenience caused to the Board and the parties.

Sincerely,

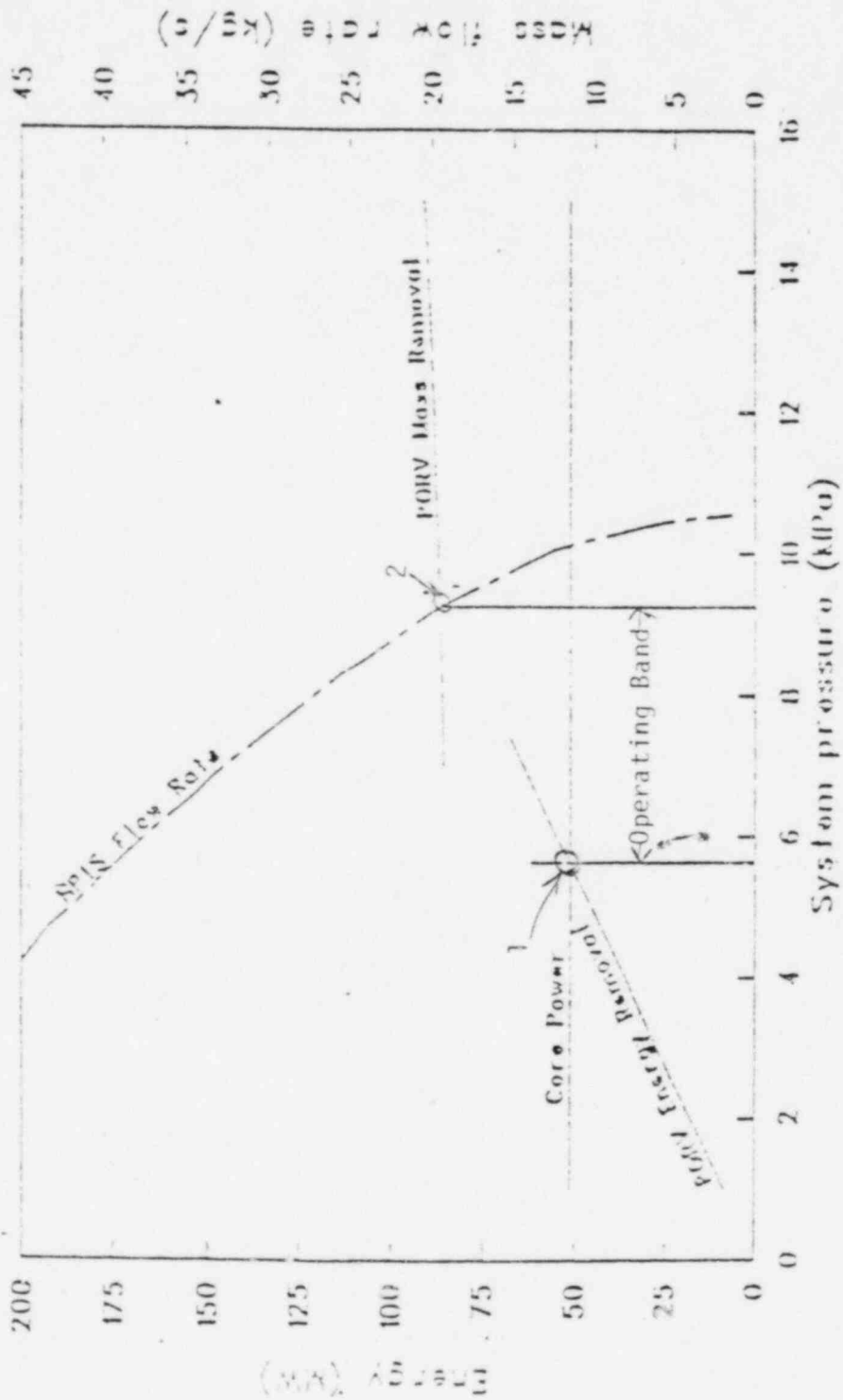
James M. Cutchin IV  
Counsel for NRC Staff

cc: Service List  
(w/ attachments)

*5507 pl*

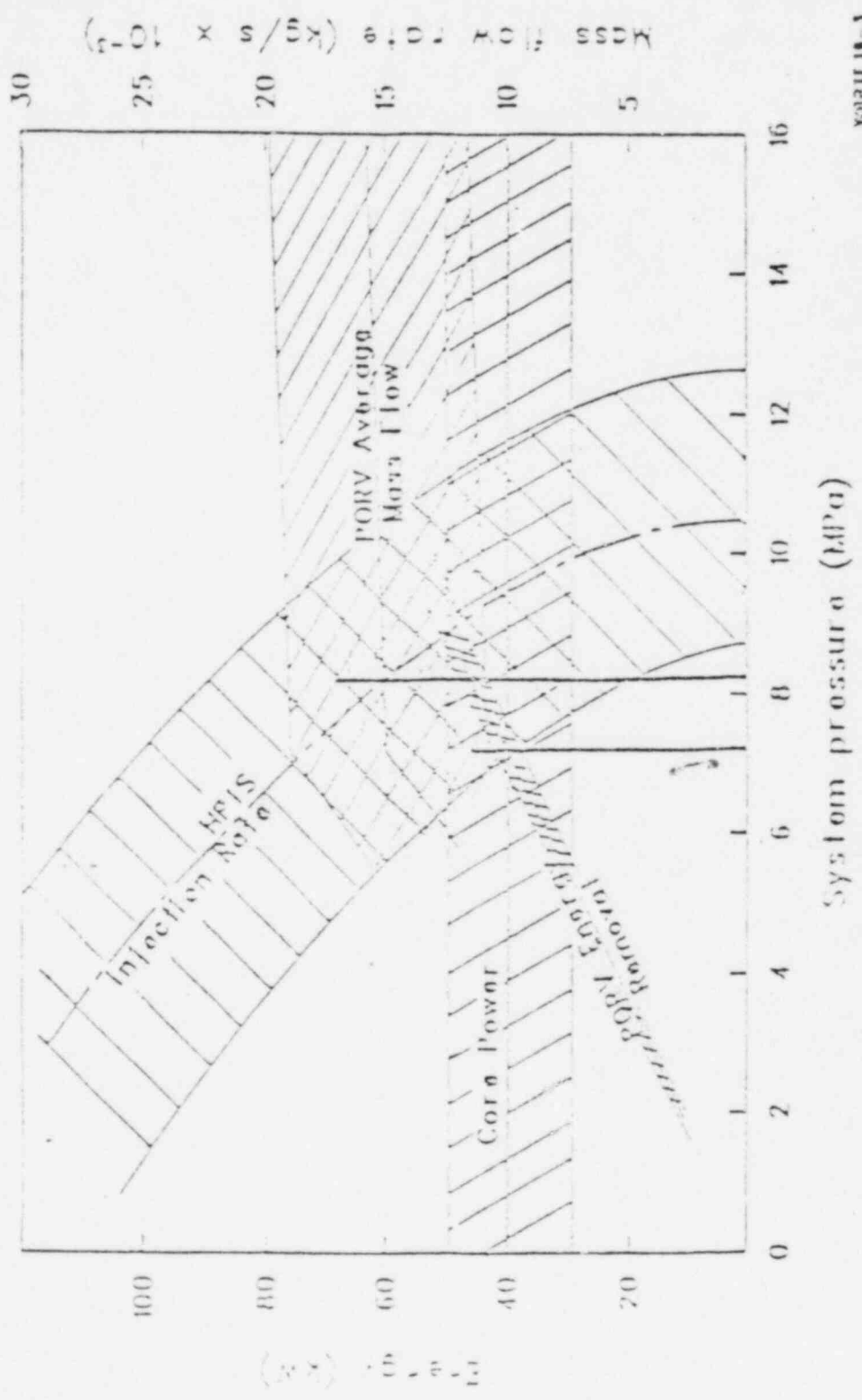
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NAME	:JCutchin:pl	:JGray	:	:	:	:	:
DATE	:11/24/82	:11/24/82	:	:	:	:	:

Figure 1



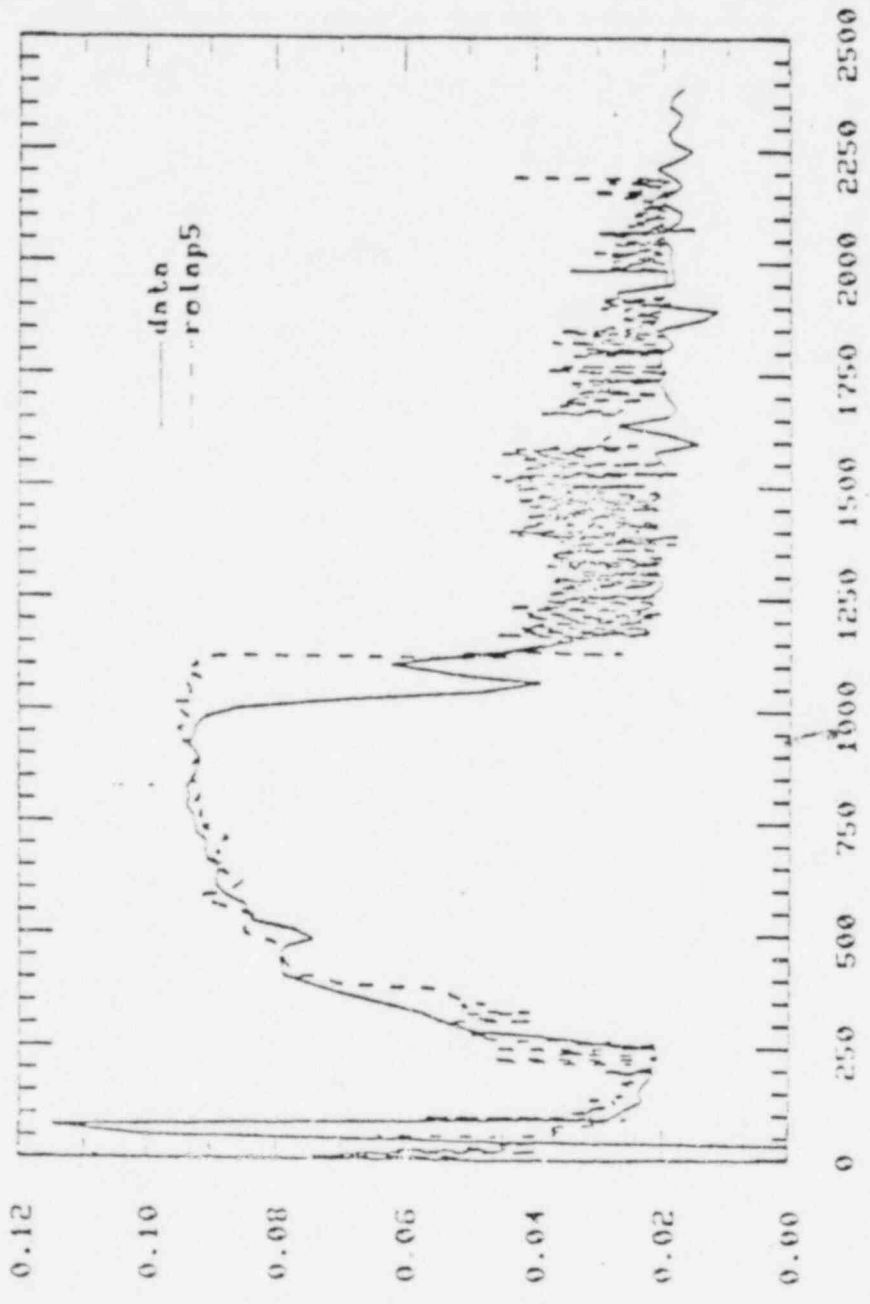
Went 17 10-6

Zion primary feed and bleed operating map for 1.5% core power without charging pumps.



sheet 16-3

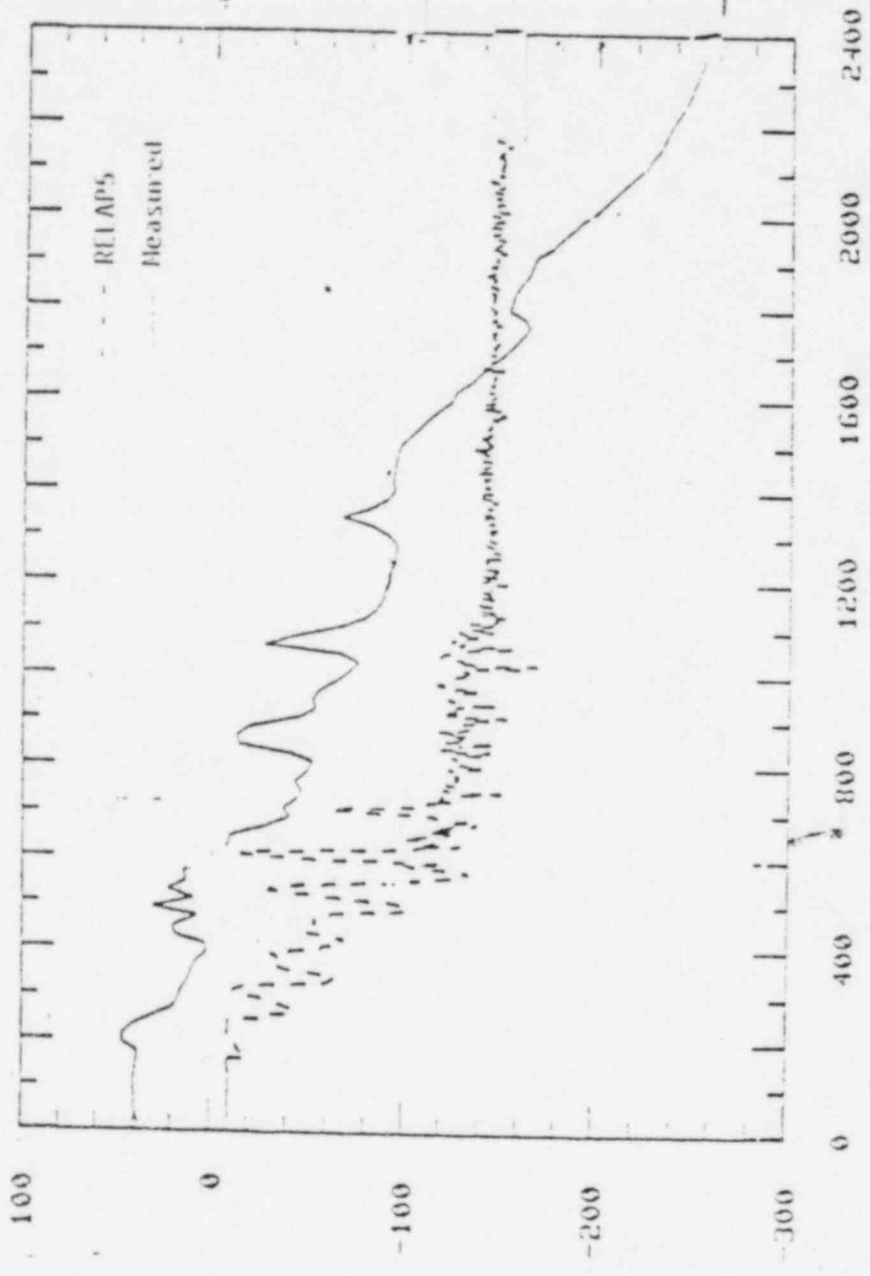
Figure 2  
 Semiscale Rod 2A primary feed and bleed operating map for 2% core  
 power with uncertainties.



Time (s)

Figure 3

Comparison of measured and RIAP's predicted PORV mass flow rate. Test S-5R-2, point 3.



Time (s)  
Figure 4

Comparison of measured and REAP5 predicted core liquid levels. Test 5-5R-2, point 3. (As originally published in EGG-SEMI-6022)

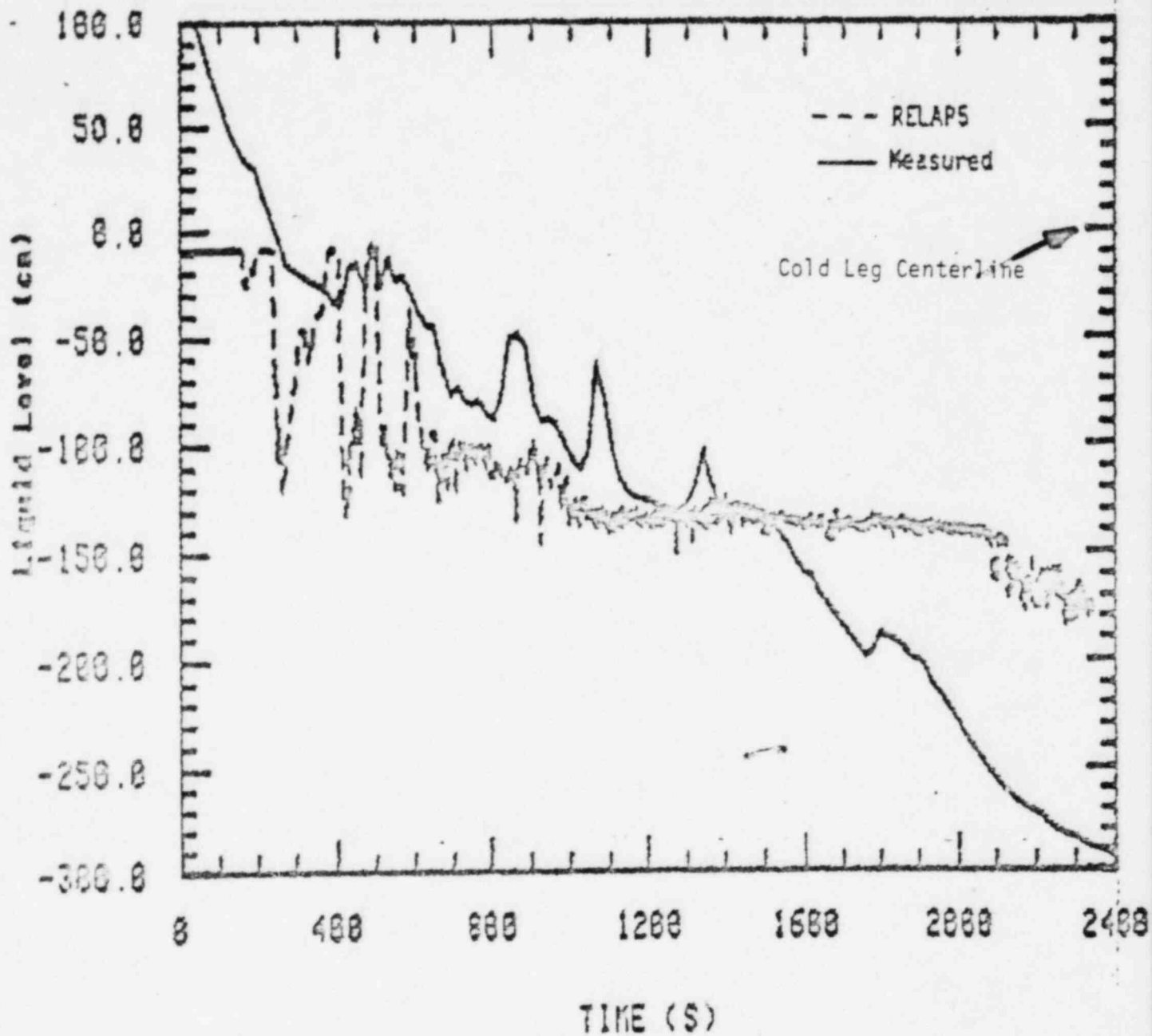
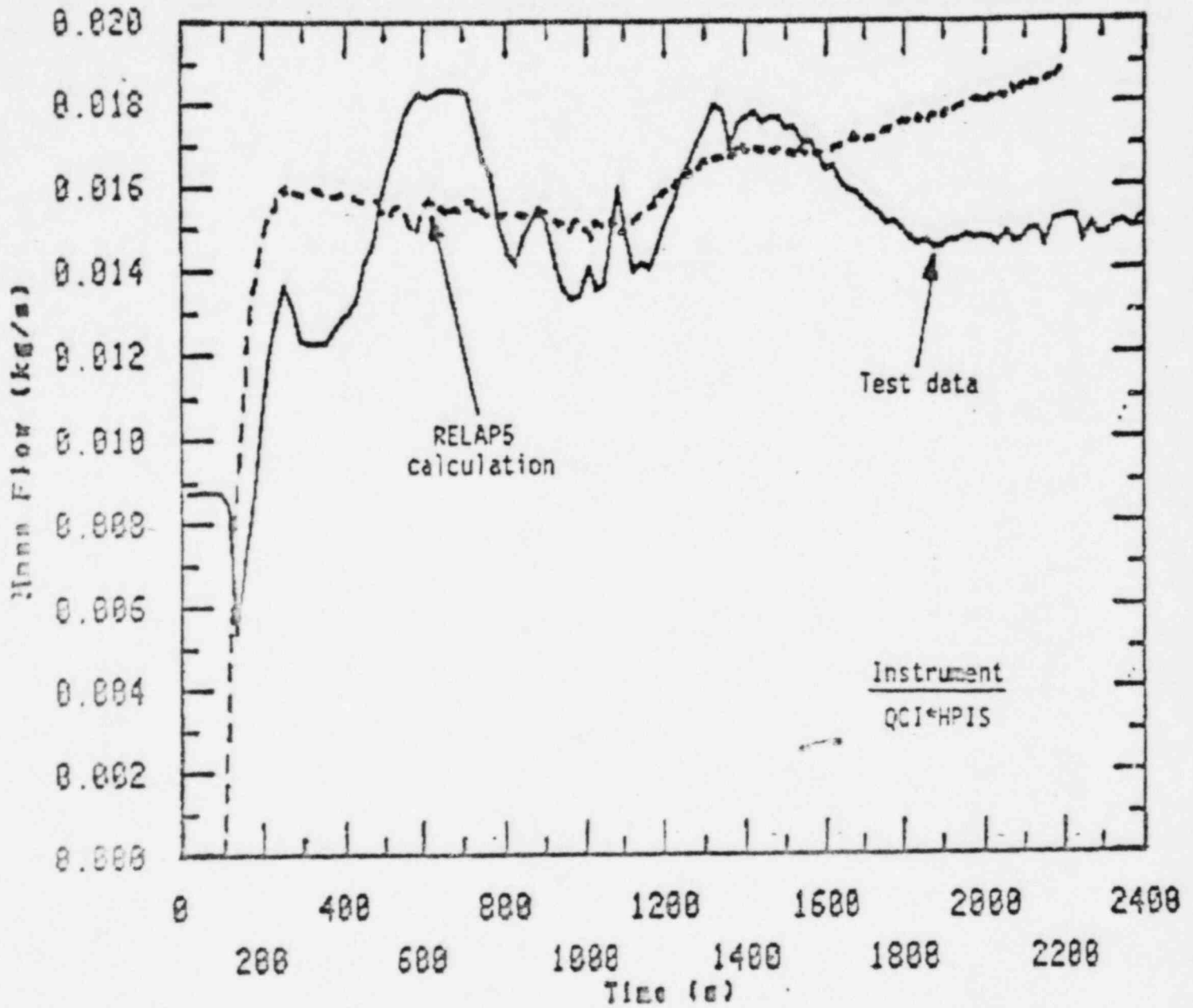


Figure 5

Comparison of measured and RELAP5 predicted vessel liquid levels. Test S-SR-2, point 3.



CEMIGALE KGD-2A TEST S-SR-2 POINT 3

Figure 6

Comparison of HPIS injection rate for RELAP5 calculation with secondary heat loss to test data. Test S-SR-2, point 3.

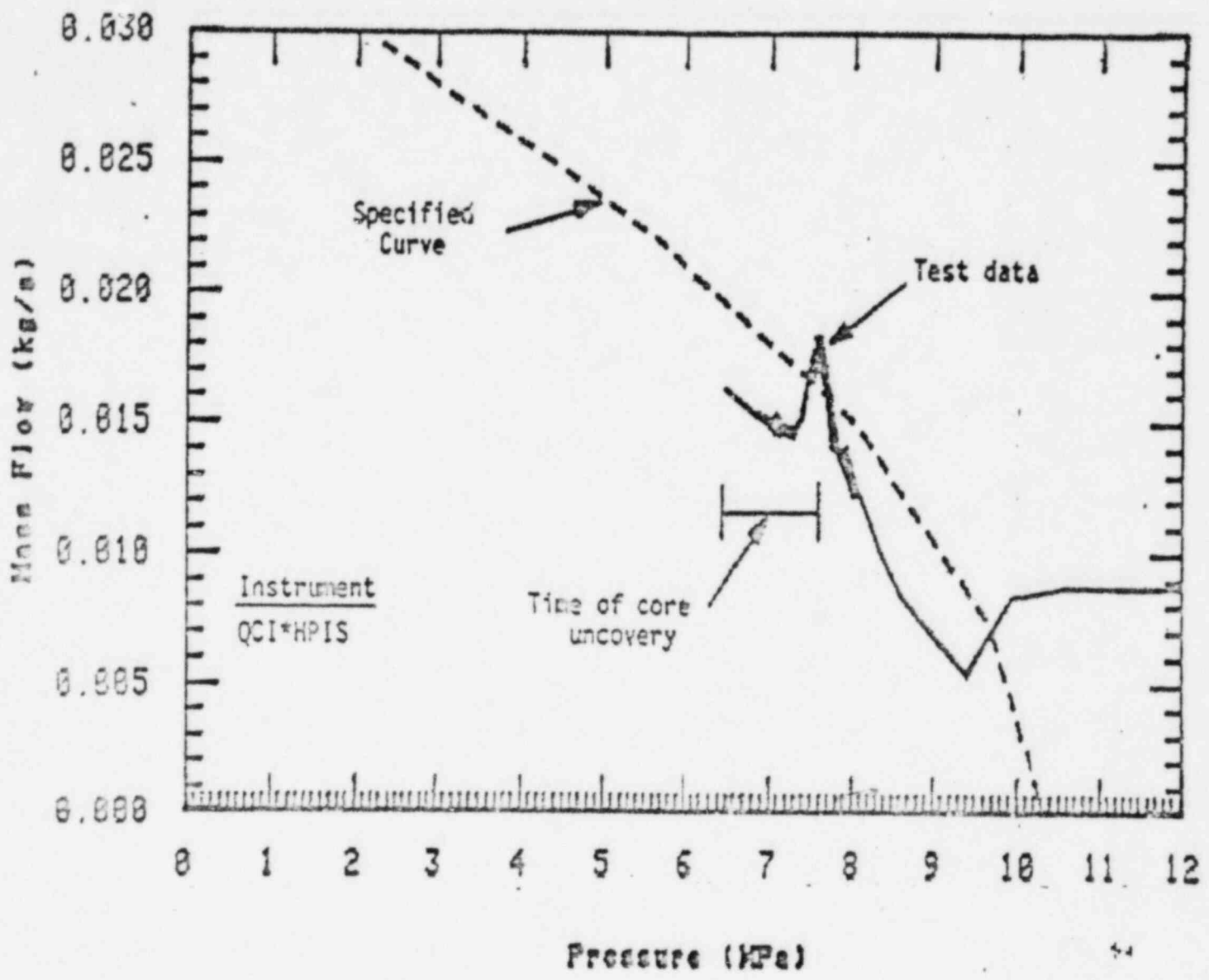


Figure 7

Comparison of actual to specified HPIS injection versus primary system pressure curves for Test S-SR-2, point 3.



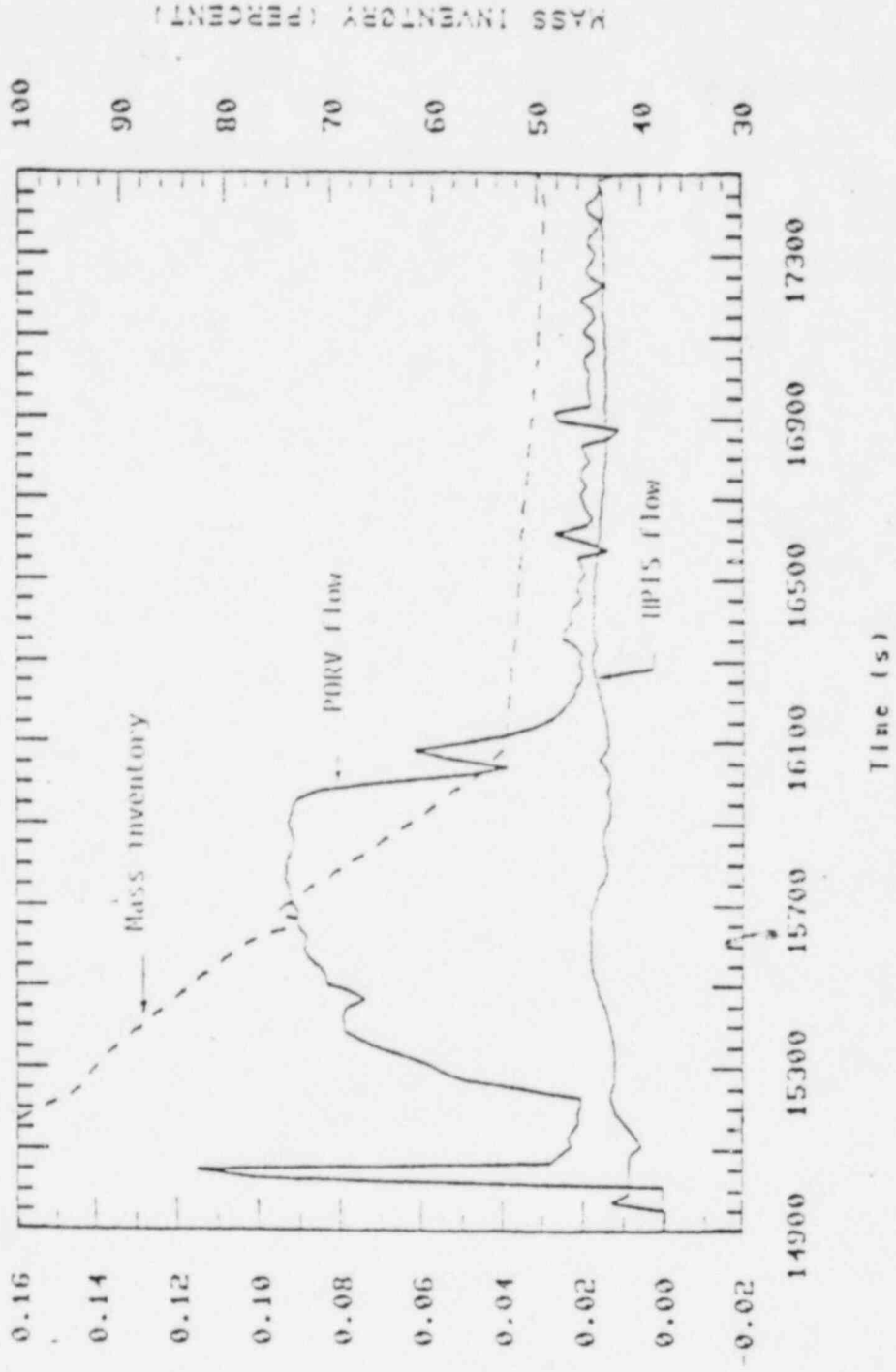


Figure 8  
 PORV and HP15 flow and primary mass inventory  
 for test S-SR-2, point 3.

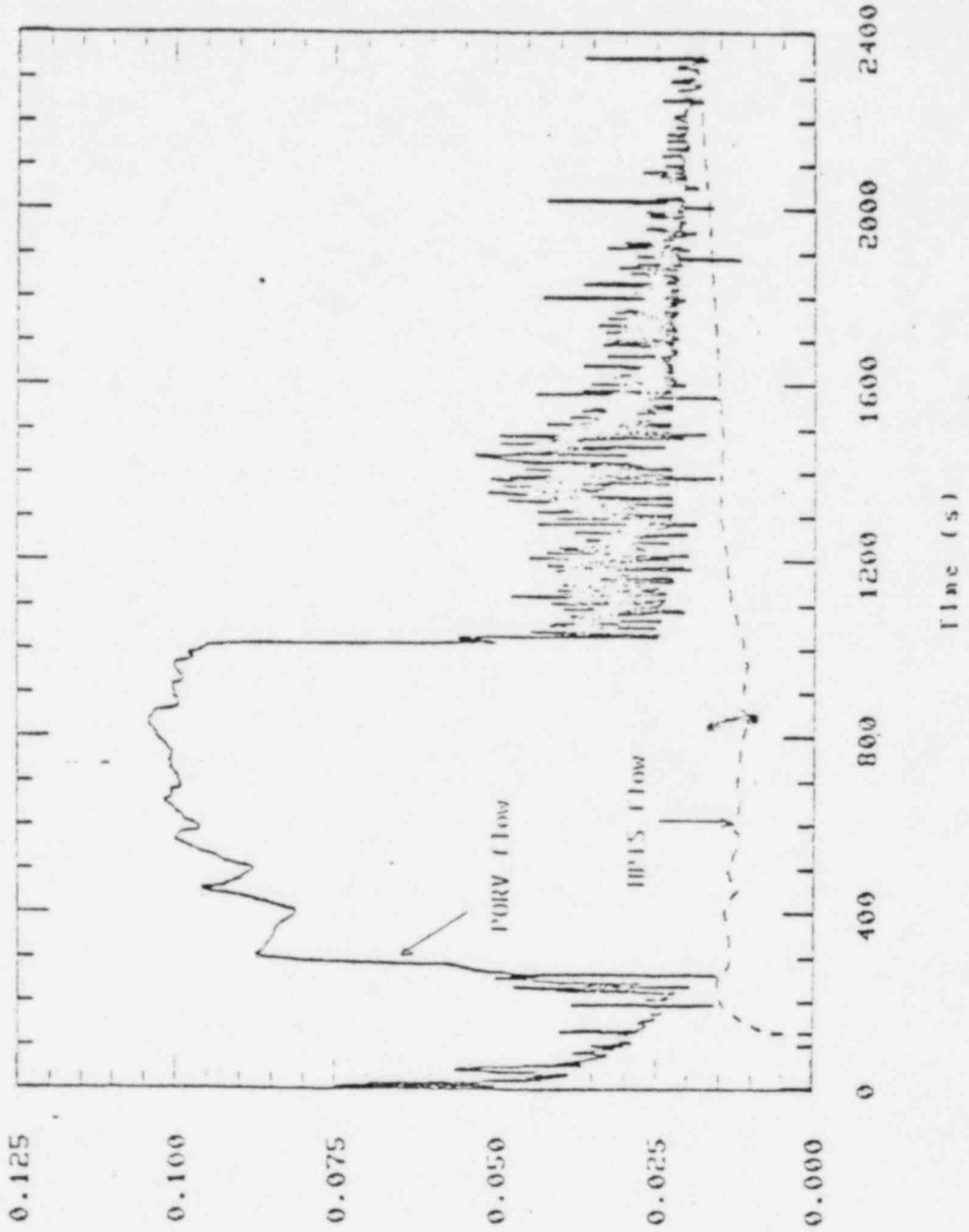
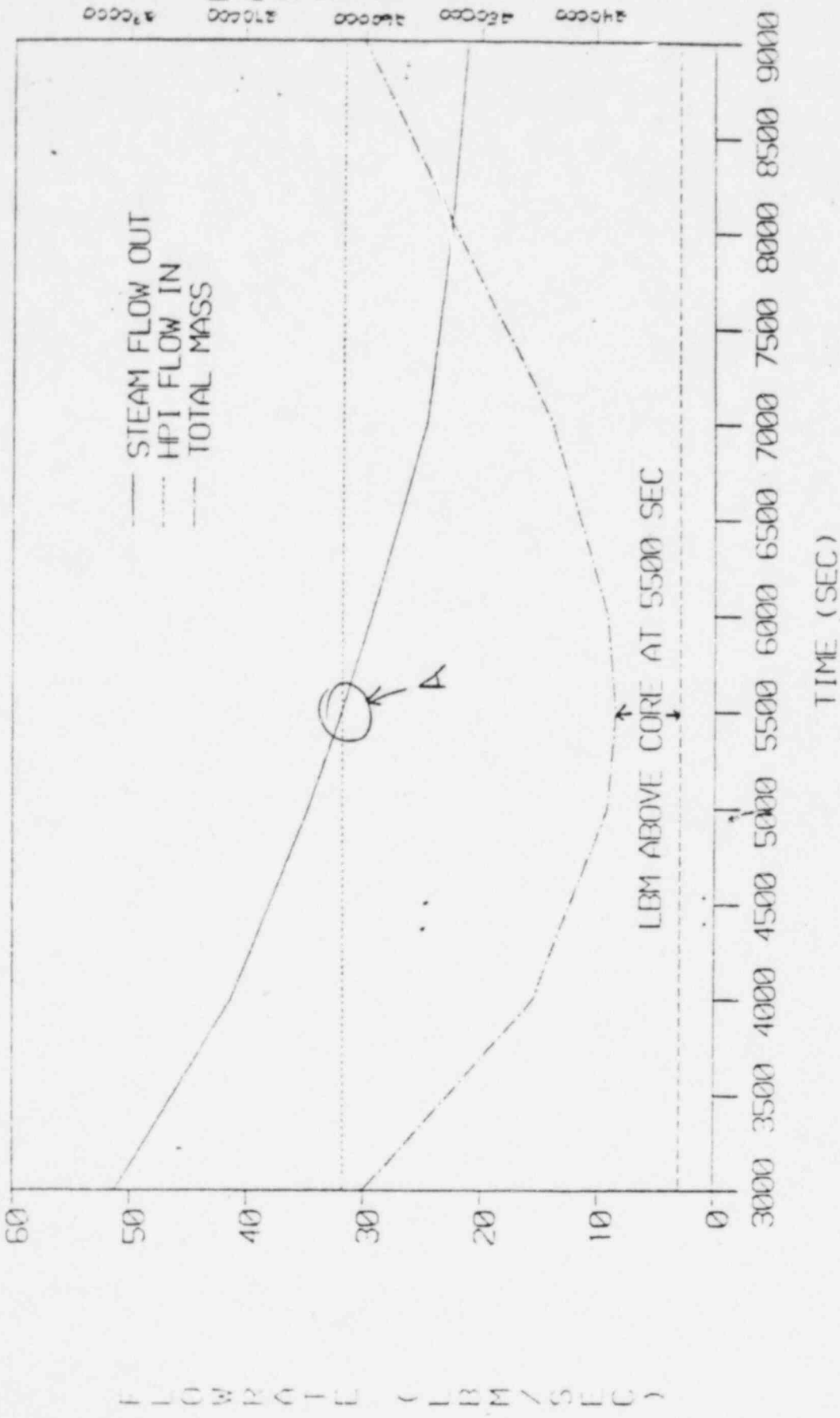


Figure 9  
 RELAP5 predicted PORV and HP15 flow rates, test 5-SR-2,  
 point 3.

PRESSURE = 2500 PSIA



400000 260000 210000 160000 110000

N C S S L B M

— STEAM FLOW OUT  
..... HPI FLOW IN  
- - - TOTAL MASS

A

LBM ABOVE CORE AT 5500 SEC

TIME (SEC)

FLOW RATE ( L B M / S E C )

# STATEMENT OF PROFESSIONAL QUALIFICATIONS

BRIAN WALTER SHERON

My name is Brian Walter Sheron. I graduated from Duke University in Durham, North Carolina, in 1969, with a Bachelor of Science in Engineering (B.S.E.) majoring in electrical engineering. I received my Masters Degree (M.S.) in nuclear engineering in 1971 and my Doctor of Philosophy (Ph.D) degree in nuclear engineering in 1975, both from the Catholic University of America in Washington, D. C.

I joined the Atomic Energy Commission in 1973 in the Division of Reactor Development and Technology and worked on the LMFBR. I joined the Nuclear Regulatory Commission in 1976 as an engineer in the Analysis Branch in the Division of Systems Safety. In 1980, I was assigned to the Reactor Systems Branch, Division of Systems Integration, and was promoted to a Section Leader in the Branch that year. In February of 1982, I was promoted to Chief of the Reactor Systems Branch. In this capacity, I supervise the activities of approximately 33 engineers in the areas assigned to the Branch.

WALTON L. JENSEN, JR.

PROFESSIONAL QUALIFICATIONS

I am a Senior Nuclear Engineer in the Reactor Systems Branch of the Nuclear Regulatory Commission. In this position I am responsible for the technical analysis and evaluation of the public health and safety aspects of reactor systems.

From June 1978 to December 1979, I was assigned to the Bulletins and Orders Task Force of the Nuclear Regulatory Commission. I participated in the preparation of NUREG-0565, "Generic Evaluation of Small Break Loss-of-Coolant Accident Behavior in Babcock & Wilcox Designed 177-FA Operating Plants."

From 1972 to 1976, I was assigned to the Containment Systems Branch of the NRC/AEC, and from 1976 to 1979, I was assigned to the Analysis Branch of the NRC. In these positions I was responsible for the development and evaluation of computer programs and techniques to calculate the reactor system and containment system response to postulated loss-of-coolant accidents.

From 1967 to 1972, I was employed by the Babcock and Wilcox Company at Lynchburg, Virginia. There I was lead engineer for the development of loss-of-coolant computer programs and the qualification of these programs by comparison with experimental data.

From 1963 to 1967, I was employed by the Atomic Energy Commission in the Division of Reactor Licensing. I assisted in the safety reviews of large power reactors, and I led the reviews of several small research reactors.

I received an M.S. degree in Nuclear Engineering at the Catholic University of America in 1968 and a B.S. degree in Nuclear Engineering at Mississippi State University in 1963.

I am a graduate of the Oak Ridge School for Reactor Technology, 1963-1964.

I am a member of the American Nuclear Society.

I am the author of three scientific papers dealing with the response of B&W reactors to Loss-of-Coolant Accidents and have authored one scientific paper dealing with containment analysis.