NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| 0 1 | LICENSEE CODE: N C B E P 1 (2) | LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) | LICENSE TYPE: 4 1 1 1 1 (4) | CAT: (5) |

CON'T

| 0 1 | REPORT SOURCE: L (6) | DOCKET NUMBER: 0 5 0 - 0 3 2 5 (7) | EVENT DATE: 1 0 2 2 8 2 (8) | REPORT DATE: 1 1 1 8 8 2 (9) |

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During plant shutdown operations, a routine comparison of suppression chamber water
0 3 level indications on the RTGB revealed a discrepancy between the narrow and wide
0 4 range level instruments. Performance of the indicator's operability test, PT-08.1.6,
0 5 revealed the suppression chamber water level was -26". While investigating the indica-
0 6 tion discrepancy the water level was then found at -32". Neither event affected the
0 7 health and safety of the public. The instrumentation failure is reported in LER
0 8 1-82-136. Technical Specifications 3.6.2.1, 6.9.1.9b

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	S H (11)	E (12)	E (13)	I N S T R U (14)	I (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	1 1 4	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	N (25)	B 0 4 C (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The first event resulted from inoperable subject RTGB instruments. The second event
1 1 occurred when 1-E11-F040 and F049 failed to fully close following reducing the level
1 2 in response to the first event. F040 and F049 were then fully closed and proper level
1 3 was reestablished. The problem affecting F040 and F049 is being reported in LER
1 4 1-82-116.

| 1 5 | FACILITY STATUS: G (28) | % POWER: 0 0 0 (29) | OTHER STATUS (30): NA | METHOD OF DISCOVERY: B (31) | DISCOVERY DESCRIPTION (32): Periodic Test |

| 1 6 | ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): NA | LOCATION OF RELEASE (36): NA |

PERSONNEL EXPOSURES

| 1 7 | NUMBER: 0 0 0 (37) | TYPE: Z (38) | DESCRIPTION (39): NA |

PERSONNEL INJURIES

| 1 8 | NUMBER: 0 0 0 (40) | DESCRIPTION (41): NA |

LOSS OF OR DAMAGE TO FACILITY (43)

| 1 9 | TYPE: Z (42) | DESCRIPTION: NA |

8211290504 821118
PDR ADOCK 05000325
S PDR

PUBLICITY

| 2 0 | ISSUED: N (44) | DESCRIPTION (45) | NRC USE ONLY |

NAME OF PREPARER M. J. Pastva, Jr. PHONE 919-457-9521

GPO 917-926

LER ATTACHMENT - RO #1-82-114

Facility: BSEP Unit No. 1 Event Date: October 22, 1982

During plant shutdown operations a routine comparison of suppression chamber water level indications on the RTGB showed a discrepancy in indications exhibited by the narrow and wide level instruments. The suppression pool level indicators operability test, PT-08.1.6, was then performed to determine the operability of the subject indicators. At that time a check of the local level sightglass revealed actual level was -26". The level was then returned to within specifications (-27" - -31") at a value of -29" as shown on the local level sightglass.

While investigating the level indications discrepancy, a check of the local level sightglass by the attending technicians revealed a level of -32". Prior to and at the time of this event the "B" loop of the RHR System had been in the suppression pool cooling mode. Following discovery of the first event, level was then returned to within specifications by opening the RHR System discharge valves to the Radwaste System, 1-E11-F040 and F049, and utilizing the already running "B" Loop RHR System to reduce suppression pool level. After attaining the proper level as indicated on the local sightglass level indicator, F040 and F049 were closed as was indicated on the RTGB. However, unknown to Operations personnel at that time, these valves did not fully close and a slow continual reduction in suppression pool level continued as long as the "B" loop RHR System was in service resulting in the level exceeding the specified lower limit. Following this discovery, F040 and F049 were declared inoperable, deactivated, and manually closed in accordance with technical specifications. The suppression pool level was then returned to within specifications. The problems affecting F040 and F049 are being reported in LER 1-82-116. In addition, instrumentation problems involving this event are being reported in LER 1-82-136.

Prior to the first event, there were no planned level changes which would have alerted the Control Operator of suppression chamber water level instrumentation inoperability. With regard to the first event, it is felt that present plant procedures which require a level instrumentation comparison prior to any planned level changes are sufficient to help prevent future similar type events. The second event may have been prevented by utilizing a more frequent visual check of the local level sightglass indicator following the discovery of a level indication discrepancy with the RTGB instruments. As a result, an entry will be made in the plant operating daily instructions to provide for utilizing a twice-per-shift visual check of the local level sightglass indicator whenever a suppression pool level indication discrepancy has been discovered and until the discrepancy is resolved. All licensed Operations personnel will review this report to ensure their familiarity with this event and the concerns outlined.