

SOUTH CAROLINA ELECTRIC & GAS COMPANY

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O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

November 19, 1982

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ATLANTA, GEORGIA
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Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street, N.W.
Atlanta, Georgia 30303

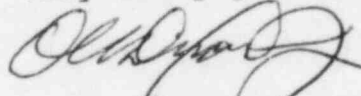
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
Thirty Day Written Report
LER 82-025

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #82-025 for Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(b) as a result of entry into Action Statement (a) of Technical Specification 3.4.4, "Reactor Coolant System, Relief Valves," on October 20, 1982.

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

ARK:OWD:dwf
Attachment

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DETAILED DESCRIPTION OF EVENT

On October 20, 1982, with the Plant in Mode 3, maintenance personnel performing Surveillance Test Procedure (STP) 127.001 found a pressurizer Power Operated Relief Valve (PORV), PCV-445A, leaking and took the valve out of service. On October 28, 1982, with the Plant in Mode 2, a second pressurizer PORV, PCV-444B, was found to be leaking and was subsequently taken out of service. On November 2, 1982, with the Plant in Mode 4 prior to cooldown below 300°F, both PORVs were put back in service for compliance with Technical Specification 3.4.9.3, "Reactor Coolant System, Overpressure Protection System." The leaking associated with the PORVs at operating temperatures and pressures had stopped when they were put back in service at this time at a lower pressure. On November 3, 1982, with the Plant in Mode 4 subsequent to exceeding 300°F the subject PORVs were again taken out of service.

PROBABLE CONSEQUENCES

No adverse consequences resulted from these events. The code safety relief valves associated with the pressurizer were operable throughout the time frame of this sequence of events.

CAUSE(S) OF THE OCCURRENCE

Steam impingement at operating temperatures and pressure caused degradation of the seat/disc interface in the PORVs.

IMMEDIATE CORRECTIVE ACTIONS TAKEN

In accordance with Action Statement (a) of Technical Specification 3.4.4, "Reactor Coolant System, Relief Valves," block valves associated with the leaking PORVs were closed and power to the motor operated block valves was removed. When the plant was cooled down for maintenance, the PORVs were returned to service in compliance with Technical Specification 3.4.9.3.

ACTION TAKEN TO PREVENT RECURRENCE

Maintenance Work Request (MWR) 04767 was initiated to effect repairs on the leaking PORVs. Additional replacement parts have been ordered in the event that leakage should occur in the future. The PORVs will be repaired as soon as scheduling and plant conditions permit.