



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

YANKEE ATOMIC ELECTRIC COMPANY

DOCKET NO. 50-29

YANKEE NUCLEAR POWER STATION (YANKEE)

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The filing by Yankee Atomic Electric Company (the licensee) dated September 17, 1981, as supplemented July 28, 1982, which is a part of the licensee's application dated July 13, 1978 (Proposed Change No. 158), as supported by letters identified in Section 1.0 of the Commission's Safety Evaluation supporting this amendment, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment. Paragraph 2.C(2) of Facility Operating License No. DPR-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.


3. In addition, Paragraph 2.C(6) is hereby added to Facility Operating License No. DPR-3 to read as follows:

(6) Storage of Spent Fuel

Notwithstanding the provisions of Section 5.6.3 of the Technical Specifications, storage of spent fuel in the spent fuel pit shall be limited to 391 fuel assemblies until the NRC staff has reviewed and accepted seismic analyses of the new spent fuel racks.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Frank J. Miraglia, Assistant Director
for Safety Assessment
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 23, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-3

DOCKET NO. 50-29

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages.* The revised pages include the captioned amendment number and contain vertical lines indicating the area of change.

PAGES

3/4 9-8

3/4 9-12

5-5

* Overleaf pages are also enclosed for maintaining document completeness.

REFUELING OPERATIONS

SHIELD TANK CAVITY

MANIPULATOR CRANE OPERABILITY

LIMITING CONDITION FOR OPERATION

3.9.6 Control rods and fuel assemblies shall be handled one-by-one with an OPERABLE shield tank cavity manipulator crane and universal handling tool with:

- a. A minimum capacity of 900 pounds, and
- b. An overload cut off limit \leq 4800 pounds.

APPLICABILITY: During movement of control rods or fuel assemblies within the reactor pressure vessel.

ACTION:

With the requirements for crane and handling tool OPERABILITY not satisfied, suspend use of the inoperable manipulator crane or handling tool from operations involving the movement of control rods and fuel assemblies within the reactor pressure vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.6 The manipulator crane and handling tool used for movement of control rods or fuel assemblies within the reactor pressure vessel shall be demonstrated OPERABLE within 100 hours prior to the start of such operations by performing a load test of at least 900 pounds, demonstrating an automatic load cut off when the crane load exceeds 4800 pounds, and verifying proper operation of the handling tool.

REFUELING OPERATIONS

CRANE TRAVEL - SPENT FUEL PIT

LIMITING CONDITION FOR OPERATION

3.9.7 Loads in excess of 900 pounds shall be prohibited from travel over the spent fuel pit except for the:

- a. Spent fuel pit building roof hatches,
- b. Spent fuel inspection stand,
- c. Fuel handling equipment,
- d. Spent fuel racks,
- e. Temporary gate, and
- f. Shielding panels.

APPLICABILITY: With fuel assemblies in the spent fuel pit.

ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.7.1 Loads in excess of 900 pounds shall be prevented from traveling over the spent fuel pit by administrative control except that the:

- a. Spent fuel pit building hatches, the spent fuel inspection stand, the fuel handling equipment, spent fuel racks, the temporary gate and the shielding panels may travel over the spent fuel pit in accordance with approved written procedures.
- b. The spent fuel inspection stand, the temporary gate and the shielding panels shall be prevented from traveling over fuel assemblies in the spent fuel pit by administrative control; and
- c. Fuel handling equipment when moved for maintenance shall be prevented from traveling over fuel assemblies in the spent fuel pit by administrative control.

4.9.7.2 Spent fuel pit building hatches, the temporary gate, and the shielding panels shall not be permitted to be moved over the spent fuel pit unless all spent fuel in the spent fuel pit has decayed for at least 90 days.

REFUELING OPERATIONS

WATER LEVEL - REACTOR VESSEL

LIMITING CONDITION FOR OPERATION

3.9.10 At least, 32 feet of water shall be maintained over the top of irradiated fuel assemblies seated within the reactor pressure vessel.

APPLICABILITY: During movement of fuel assemblies or control rods within the reactor pressure vessel while in MODE 6.

ACTION:

With the requirements of the above specification not satisfied, suspend all operations involving movement of fuel assemblies or control rods within the pressure vessel. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.10 The water level shall be determined to be at least its minimum required depth within 2 hours prior to the start of and at least once per 24 hours thereafter during movement of fuel assemblies or control rods.

REFUELING OPERATIONS

SPENT FUEL PIT WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.11.1 At least 14 feet of water shall be maintained over the top of irradiated fuel assemblies seated in the storage racks.

3.9.11.2 At least 5 feet of water shall be maintained over the top of irradiated fuel assemblies while the fuel assemblies are not seated in the storage racks.

APPLICABILITY: Whenever irradiated fuel assemblies are in the spent fuel pit.

ACTION:

With the requirements of the specification not satisfied, suspend all movement of fuel assemblies and crane operations with loads in the spent fuel pit areas and restore water level to within its limit within 4 hours. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.11 The water level in the spent fuel pit shall be determined to be at least its minimum required depth at least once per 7 days when irradiated fuel assemblies are in the spent fuel pit.

DESIGN FEATURES

DRAINAGE

5.6.2 The spent fuel storage pool is designed and shall be maintained to prevent inadvertent draining of the pool.

CAPACITY

5.6.3 The spent fuel pit is designed and shall be maintained with a storage capacity limited to no more than 721 fuel assemblies.

5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7-1.

TABLE 5.7.1

COMPONENT CYCLIC OR TRANSIENT LIMITS

<u>COMPONENT</u>	<u>CYCLIC OR TRANSIENT LIMIT</u>	<u>DESIGN CYCLE OR TRANSIENT</u>
Reactor Pressure Vessel	200 heatup and cooldown cycles > 50°F/hour and <u>≤</u> 100°F/hour.	70°F to 534°F to 70°F