NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77) Attachment to AECM-82/558 LICENSEE EVENT REPORT Page 1 of 2 CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL AEQUIRED INFORMATION) 00000 01 G S CON'T REPORT (6)0 5 0 0 4 1 6 (7) 1 0 2 1 8 2 (8) 1 0 10 SOURCE DOCKET NUMBER EVENT DATE REPORT DATE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On October 21, 1982, during hot functional testing of the NSSS, Plant Service Water 0 2 (PSW) was lost for approximately fifteen (15) minutes. As a result, cooling water flow 0 to the drywell coolers was interrupted. Drywell average temperature exceeded 135°F, 0 4 and an LCO was entered under T.S.3.6.2.6. Cooling was restored, and drywell 5 0 temperature decreased to less than 135°F. (See attached supplement). This event had 0 6 no effect on the health and safety of the public and did not constitute a threat to plant safety. 80 CODE COMP CAUSE CAUSE VALVE COMPONENT CODE SUBCODE SUBCODE Z (13) S | B IX I Z ZI Z 1 ZIZI Z (14) Z 1(15 Z1 (16 SEQUENTIAL OCCURRENCE REVISION REPORT REPORT NO LER/RO REPORT EVENT YEAR TYPE NO. 0 0 L 4 NUMBER ATTACHMENT SUBMITTED NPHD-4 PRIME COMP COMPONENT ACTION FUTURE EFFECT ON PLANT SHUTDOWN METHOD (22) HOURS MANUFACTURER FORM SUB. SUPPLIER 9 (26) 2 9 9 0 0 Y 0 0 N Z (23) (25 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 PSW was lost as a result of a spurious trip of the circuit breaker feeding the plant Iradial well pumps. The breaker was reset, and cooling water was restored to the |drywell within fifteen (15) minutes. Subsequent investigations revealed no circuit This breaker malfunction. The cause of the trip could not be specifically 'stermined. is a final report. 4 80 METHOD OF DISCOVERY FACILITY (30) DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER Annunciator Actuation (28) 01 0 A (31) B 0 NΛ 4.4 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED OF RELEASE Z (34) NA Z (33) NA 80 44 PERSONNEL EXPOSURIES DESCRIPTION (39) 0 0 0 (37) Z (38) NA 80 PERSONNEL INJUP ES JESCRIPTION (41) NUMBER 0 0 (40) NA 0 80 8211290350 821119 PDR ADOCK 05000416 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION PDR Z (42) NA 80 PUBLICITY NRC USE ONLY DESCRIPTION (45) SUED N (44) NA 69 80 NAME OF PREPARER Original signed by Ken Walker PHONE -

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SUPPLEMENTARY INFORMATION TO LER 82-124/03 L-0

Mississippi Power & Light Company Grand Gulf Nuclear Station - Unit 1 Docket No. 50-416

Technical Specification Involved: 3.6.2.6 Reported Under Technical Specification: 6.9.1.13.b

## Event Narrative:

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On October 21, 1982, during hot functional testing of the Nuclear Steam Supply System, Plant Service Water (PSW) was lost due to the trip of a circuit breaker supplying power to the radial well pumps. Efforts were immediately initiated to line up Standby Service Water to the drywell coolers and to recover power to the PSW pumps. During recovery actions drywell temperature, as measured by averaging the required drywell temperature elements, reached 138°F. An LCO was entered under Technical Specification 3.6.2.6, which requires drywell temperature to remain below 135°F. Cooling was re-established with PSW within approximately fifteen (15) minutes. Drywell temperature guickly decreased to less than 135°F. Drywell temperature exceeded 135°F for approximately twenty-two (22) minutes. No circuit breaker trip "flags" were found and no circuit breaker malfunctions could be identified. Subsequent observation of the breaker has revealed no further problems.

Drywell temperature levels did not exceed environmental qualification limits of any equipment in this area. This is intended as a final report.

Previous Similar Events:

None