

INSTRUCTIONS FOR FILING AMENDMENT NO. 9

Pages 12.2-2 is reprinted one-sided to correct a page assembly error in Amendment 4. Pages 19.3.3.32-1/19.3.3.32-2 and 19.3.3.33-1/19.3.3.33-2 are included because they were omitted from Revision 8. Tabs are also included for Appendices 18A and 18B. *

Remove and insert the pages listed below. Dashes (----) in the remove or insert column indicate no action required.

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New Chapter 18

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10.3 MAIN STEAM SUPPLY

10.3.1 Design Bases

Applicant to provide.

10.3.2 Description

The main steam lines between the reactor pressure vessel and the shutoff valves are described in Section 5.4. Section 5.2 discusses in-service inspection, materials, and environmental conditions for the safety-related portions of the system piping. A discussion of the measures provided to limit blowdown of the system in the event of a steam line break is presented in Subsection 5.4.4.

The remainder of the description for the main steam supply will be supplied by the Applicant.

10.3.3 System Evaluation

The portion of the main steam supply described in this section has no safety-related function as discussed in Section 3.2, and therefore is not classified as Seismic Category I.

10.3.4 Tests and Inspections

Inspection and testing will be carried out in accordance with the requirements of Table 3.2-1. The main steam line will be hydrostatically tested to confirm leak tightness. Visual inspection of the pipe weld joints in the main steam line will confirm the exterior condition of the weld.

10.1

10.3.5 Water Chemistry (PWR)

Not applicable to BWR.

10.3.6 Steam and Feedwater System Materials

Section 5.4 discusses the Class 2 and 3 portion of the main steam and feedwater piping and contains a discussion of materials.

10.1

12.2.1.2 Reactor Building

12.2.1.2.1 Reactor Vessel Sources

12.2.1.2.1.1 Radiation from the Reactor Core

12.2.1.2.1.1.1 General

The information in this section defines a reactor vessel model and the associated gamma and neutron radiation sources. This section is designed to provide the data required for calculations beyond the vessel. The data selected were not chosen for any given program, but were chosen to provide information for any of several shield program types. In addition to the source data, calculated radiation dose levels are provided at locations surrounding the vessel. These data are given as a potential check point for calculations by shield designers.

12.2.1.2.1.1.2 Physical Data

Table 12.2-1 presents the physical data required to form the model in Figure 12.2-1. This model was selected to contain as few separate regions as possible to adequately portray the reactor. Table 12.2-1 provides nominal dimensions and material volume fractions for each boundary and region in the reactor model. To describe the reactor core, Table 12.2-1 provides thermal power, power density, core dimensions, core average material volume fractions and reactor power distributions. The reactor power distributions are given for both radial and axial distributions. These data contain uncertainties in the volume regions near the edge of the core. The level of uncertainties for these regions is estimated at 20%.

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18.1 HUMAN FACTORS ENGINEERING TEAM

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18.5 CONTROL CENTERS OUTSIDE OF THE MAIN CONTROL ROOM

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18B.1 SUMMARY

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18B.2 GENERAL DESCRIPTION (continued)

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18B.4 GRAPHIC DISPLAYS (continued)

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18B.4 GRAPHIC DISPLAYS (continued)

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18B.4 GRAPHIC DISPLAYS (continued)

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18B.4 GRAPHIC DISPLAYS (continued)

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18B.4 GRAPHIC DISPLAYS (continued)

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18B.5 CONCLUSION (continued)

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Figure 18B-1. Critical Plant Variable Display

18B-12

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Figure 18B-2. RPV Control Display

18B-13/18B-14

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Figure 18B-3. Containment Control Display

19.3.3.32 QUESTION/RESPONSE 3.32 (241.23)

QUESTION 3.32

Provide details of the interpolation control number scheme used for the solution in FLUSH. (3A.3)

RESPONSE 3.32

The details of the interpolation control number scheme used for the solution in FLUSH were provided during the review of PDA GESSAR.]

19.3.3.33 QUESTION/RESPONSE 3.33 (241.24)

QUESTION 3.33

For vertical SSI analysis, the depth of the water table governs the compressional wave velocity in the medium. Reevaluate your results for vertical analysis taking into account the effect of the depth of the water table. (3A.5.2)

RESPONSE 3.33

The effect of the depth of the water table on response was accounted for by a variation in the modulus values for both PDA GESSAR and GESSAR II.]