



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-325

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51  
License No. DPR-71

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Carolina Power & Light Company dated September 29, 1982 and August 25, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-71 is hereby amended by changing paragraphs 2.A and 2.C.(2) to read as follows:
  - 2.A This license applies to the Brunswick Steam Electric Plant, Unit 1, a boiling water reactor and associated equipment (the facility), owned by the Carolina Power & Light Company and North Carolina Eastern Municipal Power Agency and operated by Carolina Power & Light Company. The facility is located on the Cape

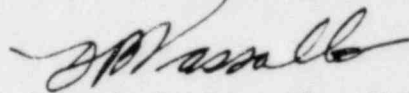
Fear River, near Southport in Brunswick County, North Carolina, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 31) and the "Environmental Report" as supplemented and amended.

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 51 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 51

FACILITY OPERATING LICENSE NO. DPR-71

DOCKET NO. 50-325

Revise Appendix A by removing page 3/4 1-4 and replacing with identically numbered page.

## REACTIVITY CONTROL SYSTEMS

### LIMITING CONDITION FOR OPERATION (Continued)

#### ACTION: (Continued)

2. If the inoperable control rod(s) is inserted:
  - a) Within one hour disarm the associated directional control valves either:
    - 1) Electrically, or
    - 2) Hydraulically by closing the drive water and exhaust water isolation valves.
  - b) Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
- c. With more than 8 control rods inoperable, be in at least HOT SHUTDOWN within 12 hours.

#### SURVEILLANCE REQUIREMENTS

4.1.3.1.1 The scram discharge volume drain and vent valves shall be demonstrated OPERABLE at least once per 31 days by:\*

- a. Verifying each valve to be open.
- b. Cycling each valve at least one complete cycle of full travel.

4.1.3.1.2 All withdrawn control rods not required to have their directional control valves disarmed electrically or hydraulically shall be demonstrated OPERABLE by moving each control rod at least one notch:

- a. At least once per 7 days when above the preset power level of the RWM and RSCS, and
- b. At least once per 24 hours when above the preset power level of the RWM and RSCS and any control rod is immovable as a result of excessive friction or mechanical interference.

4.1.3.1.3 All withdrawn control rods shall be determined OPERABLE by demonstrating the scram discharge volume drain and vent valves OPERABLE, when the reactor protection system logic is tested per Specification 4.3.1.2, by verifying that the drain and vent valves:

- a. Close within 30 seconds after receipt of a signal for control rods to scram, and
- b. Open when the scram signal is reset or the scram discharge volume trip is bypassed.

\*These valves may be closed intermittently for testing under administrative control.



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CAROLINA POWER & LIGHT COMPANY

DOCKET NO. 50-324

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 76  
License No. DPR-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by Carolina Power & Light Company dated September 29, 1982 and August 25, 1982 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-62 is hereby amended by changing paragraphs 2.A and 2.C.(2) to read as follows:
  - 2.A This license applies to Brunswick Steam Electric Plant Unit 2, a boiling water reactor and associated equipment (the facility), owned by the Carolina Power & Light Company and North Carolina Eastern Municipal Power Agency and operated by Carolina Power & Light Company. The facility is located on the Cape Fear River, near Southport in Brunswick County, North Carolina, and is described in the "Final Safety Analysis Report" as supplemented and amended (Amendments 1 through 29)

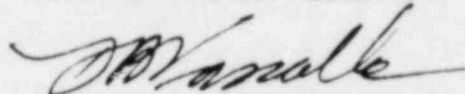
and the "Environmental Report" as supplemented and amended (Supplements 1 through 7).

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 76, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 12, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 76

FACILITY OPERATING LICENSE NO. DPR-62

DOCKET NO. 50-324

Remove the following pages and replace with identically numbered pages.

2-4, 3/4 1-4

TABLE 2.2.1-1

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS

<u>FUNCTIONAL UNIT AND INSTRUMENT NUMBER</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Intermediate Range Monitor, Neutron Flux - High <sup>(1)</sup> (C51-IRM-K601A,B,C,D,E,F,G,H)	$\leq$ 120 divisions of full scale	$\leq$ 120 divisions of full scale
2. Average Power Range Monitor (C51-APRM-CH.A,B,C,D,E,F)		
a. Neutron Flux - High, 15% <sup>(2)</sup>	$\leq$ 15% of RATED THERMAL POWER	$\leq$ 15% of RATED THERMAL POWER
b. Flow-Biased Neutron Flux - High <sup>(3)(4)</sup>	$\leq$ (0.66 W + 54%)	$\leq$ (0.66 W + 54%)
c. Fixed Neutron Flux - High <sup>(4)</sup>	$\leq$ 120% of RATED THERMAL POWER	$\leq$ 120% of RATED THERMAL POWER
3. Reactor Vessel Steam Dome Pressure - High (B21-PS-N023A,B,C,D)	$\leq$ 1045 psig	$\leq$ 1045 psig
4. Reactor Vessel Water Level - Low, Level #1 (B21-LIS-N017A,B,C,D)	$>$ 162.5 inches above instrument zero	$>$ 162.5 inches above top fuel guide
5. Main Steam Line Isolation Valve - Closure <sup>(5)</sup> (B21-FO22A,B,C,D; B21-FO28A,B,C,D)	$\leq$ 10% closed	$\leq$ 10% closed
6. Main Steam Line Radiation - High (D12-RM-K603A,B,C,D)	$\leq$ 3 x full power background	$\leq$ 3.5 x full power background
7. Drywell Pressure - High (C72-PS-N002A,B,C,D)	$\leq$ 2 psig	$\leq$ 2 psig
8. Scram Discharge Volume Water Level - High (C12-LSH-N013A,B,C,D) (C12-LSH-4516A,B,C,D)	$\leq$ 109 gallons	$\leq$ 109 gallons



## REACTIVITY CONTROL SYSTEMS

### LIMITING CONDITION FOR OPERATION (Continued)

#### ACTION: (Continued)

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  - a) Within one hour disarm the associated directional control valves either:
    - 1) Electrically, or
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  - b) Otherwise, be in at least HOT SHUTDOWN within the next 12 hours.
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