

Commonwealth Edison LaSalle County Nuclear Station Rural Route #1, Box 220 Marseilles, Illinois 61341 Telephone 815/357-6761

January 24, 1991

Director of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Mail Station P1-137 Washington, D.C. 20555

Dear Sir:

Licensee Event Report \$90-014-00, Docket \$050-373 is being submitted to your office in accordance with 10CFR50.73(a)(2)(v).

G. J. Diederich Station Manager LaSalle County Station

GJD/HTV/kg

Enclosure

xc: Nuclear Licensing Administrator NRC Resident Inspector NRC Region III Administrator INPO - Records Center

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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 25, 1990 at 1140 hours, with Unit 1 in Operational Condition 1 (Run) at 97% power, the average electrolyte temperature of the Unit 1 250V battery was found to be 62 degrees F. Although there are no Technical Specification temperature requirements on this battery, LaSalle Operating Surveillance LOS-AA-D1 "Unit Daily Surveillances" requires the Unit 1 250V battery to be declared inoperable should average electrolyte temperature fall below 65 degrees F. Since this battery also provides emergency power to the Reactor Core Isolation Cooling (RCIC) system, the RCIC system was also declared inoperable and a 14-day timeclock initiated in accordance with Technical Specifications.

The apparent cause of this event appears to be the inability of the Unit 1 Division I Switchgear Heat Removal System (VX) temperature controller to maintain proper temperature in the 250V battery room due to outside seasonal temperature changes.

The consequences of this event were minimal. The High Pressure Core Spray system, which is the alternate high pressure injection system, was fully operable throughout this event. In addition, the Automatic Depressurization System and all Emergency Core Cooling System (ECCS) low pressure injection systems were fully operable.

Initial corrective actions were to restore the Division I switchgear room to normal temperature. This was accomplished by locating portable heaters in the switchgear room to increase the heat input to the room and the failed outside ventilation air damper was closed to reduce the admission of cool outside air. These actions quickly restored room temperature and battery electrolyte temperature to normal within 24 hours. The RCIC system was then declared operable and the Technical Specification timeclock exited on December 26 at 0345 hours.

This event is reported pursuant to IOCFR50.73(a)(2)(v) due to the loss of a safety function system.

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TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

#### PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): <u>1</u> .	Event Date: 12/25/90	Event	Time: Hours	
Reactor Mode(s):	Mode(s) Name:	RUN	Power Level(s): _9	7%

# B. DESCRIPTION OF EVENT

On December 25, 1990 at 1140 hours, with Unit 1 in Operational Condition 1 (Run) at 97% power, the average electrolyte temperature of the Unit 1 250V battery (DC)[EJ] was found to be 62 degrees F. Battery temperatures were being taken in accordance with LaSalle Operating Surveillance LOS-DC-Q2, "Safety-Related Battery Equalizing Charges". Although there are no Technical Specification temperature requirements on this battery, LaSalle Operating Surveillance LOS-AA-D1 "Unit Daily Surveillances" requires the Unit 1 250V battery to be declared inoperable should average electrolyte temperature fall below 65 degrees F. Since this battery also provides amergency power to the Reactor Core Isolation Cooling system (RCIC)(RI)[BN], the RCIC system was also declared inoperable and a 14-day timeclock initiated in accordance with Technical Specification 3.7.3. Work Request L03979 was initiated to resolve the problem.

This event is reportable pursuant to 10CFR50.73(a)(2)(v) due to a condition that could have prevented the fulfillment of a safety function of RCIC if needed to remove residual heat.

## C. APPARENT CAUSE OF EVENT

The cause of this event was battery electrolyte temperature falling below its minimum calculated acceptable value (with respect to battery capacity). This condition necessitated the RCIC system being declared inoperable and a Technical Specification timeclock entered.

The apparent cause of this event appears to be the inability of the Unit 1 Division I Switchgear Heat Removal System (VX)[VF] temperature controller to maintain proper temperature in the 250V battery room due to outside seasonal temperature changes.

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TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]

### D. SAFETY ANALYSIS OF EVENT

The consequences of this event were minimal. The High Pressure Core Spray system (HPCS)(HP)[BG] which is the alternate high pressure injection system, was fully operable throughout this event. In addition, the Automatic Depressurization System (ADS)(NB)[SB] and all Emergency Core Cooling System (ECCS) low pressure injection systems were fully operable.

Although no Technical Specifications address electrolyte temperature of the Unit 1 250V battery, previous calculations per the LaSalle Electrical Load Monitoring System (ELMS) indicates the capacity of the battery is insufficient to satisfy the requirements of the Updated Final Safety Analysis Report (UFSAR) with electrolyte temperature below 6E degrees F. Since electrolyte temperature was 62 degrees F, substantial battery capacity still remained to provide emergency power to the RCIC system if needed.

Division I Switchgear Room temperatures were verified satisfactory per LOS-AA-D1 approximately B hours prior to this event.

### E. CORRECTIVE ACTIONS

Initial corrective actions were to restore the Division I switchgear room to normal temperature. This was accomplished two-fold: 1) portable heaters were located in the switchgear room to increase the heat input to the room, and 2) the failed outside ventilation air damper was wired closed (per an approved Temporary System Change) to reduce the admission of cool outside air. These actions quicily restored room temperature and battery electrolyte temperature to normal within 24 hours. The RCIF system was then declared operable and the Technical Specification timeclock exited at 0345 hours or December 26, 1990.

A design deficiency has been identified with the temperature controllers which position the VX system dampers. This problem will be resolved per a design change to install controllers capable of better responding to outside seasonal temperature changes. Although this event was specifically related to Division I VX, this corrective action will also be applied to Division II VX since it currently utilizes the same design. Action Item Record (AIR) 373-200-90-09102 will track the completion of these design changes.

As an interim measure, a surveillance to seasonally position the VX outside air dampers will be generated and placed in the FaSalle surveillance program. AIR 373-200-90-09101 will track this action.

The Unit 1 250V battery is scheruled to be replaced during the Unit 1 fourth refueling outage. This outage is currently scheduled for February 1991. The battery is being replaced with a larger capacity battery in compliance with 10 CFR Part 50.63 (Station Blackout). Following the replacement, the larger capacity battery will allow the battery to remain operable to 37 degrees F.

#### F. PREVIOUS EVENTS

LER Number

Title

374/89-018-00 Reactor Core Isolation Cooling System Inoperable due to 250 vdc Battery Low Temperatures Caused by Failed Damper and Low Outside Air Temperatures

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w.,	COMPONENT FAILURE DATA											
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Temperature Controller N/A

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