SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO CHANGE IN CHAPTER 14 FSAR, STARTUP TEST PROGRAM

NIAGRARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT 2

DOCKET NO. 50-410

1.0 Introduction

By letter dated September 9, 1987, Niagara Mohawk Power Corporation (the licensee) submitted changes made to their initial startup test program. The submittal was made in accordance with license condition C(8) which requires that any changes to the initial startup test program described in Section 14 of the FSAR be made in accordance with the provision of 10 CFR 50.59 and reported, in accordance with 50.59(b), within one month of such change.

2.0 Description

The letter detailed two changes:

- (1) The requirement for RCIC (Reactor Core Isolation Cooling) pump discharge pressure during CST (Condensate Storage Tank) injection testing was changed from 100 (+20,-0) psi above reactor pressure to > 100 psi above reactor pressure.
- (2) The requirement to test the RWCU (Reactor Water Cleanup) system during test condition heatup at rated reactor pressure and temperature and in both its normal and hot standby (startup) modes was changed to require testing in the startup alignment and in the normal operating alignment. In addition, the requirement to calibrate the bottom head flow indicator and to measure pump vibration at the coupling end were deleted.

3.0 Reference

Regulatory Guide 1.68, Initial Test Programs for Water-cooled Nuclear Power Plants, Revision 2, August, 1978.

4.0 Evaluation

(1) RCIC Pump Test Pressure

To simulate expected operating conditions during Reactor Core Isolation Cooling - Condensate Storage Tank (RCIC-CST) injection test, the pump discharge pressure must exceed the reactor pressure by an amount that equals or exceeds the maximum pipeline pressure loss during actual vessel injection. The expected maximum pipeline pressure loss during actual vessel injection is 100 psi.

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9101290211 901016 PDR FDIA PERSON90-269 PDR The change removes the upper tolerance limit and permits testing at higher pump discharge pressure which subjects the system to a more stringent test and therefore, bounds expected operational characteristics. The change is therefore found acceptable.

(2) RWCU System Test

The change in wording does not change the modes of operation in which the system will be tested. The removal of the requirement to conduct these tests during test condition heatup (low power) is necessary since plant specific restraints do not allow placing the RWCU system in its normal alignment until reactor power exceed 20% of rated power. The change is therefore acceptable.

The deletion of the requirement to calibrate the bottom head flow indicator is acceptable since this calibration was accomplished during the preoperational test program.

The deletion of the requirement to measure pump vibration at the coupling end is acceptable due to the pump vibration measurements made on the bearing housing and shaft which provide adequate assurance of acceptable pump performance.

5.0 Conclusion

We have concluded, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations; and (3) the issuance of this change will not be inimical to the common defense and security or to the health and safety of the public.

Date:

Principal Contributor: L. J. Wink

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SALP INPUT

Docket No.: 50-410

Facility: Nine Mile Point Unit 2

TAC No.: 66179

Requested Date: September 9, 1987

Functional Area: Licensing Activity

Region I Reviewer: L. Wink

SALP Input:

The licensee's request to make changes to the FSAR to reflect changes in their initial startup test program was found to be technically adequate. The bases provided to justify this change was clear and acceptable.

Conclusion:

Category I.

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V NIAGARA

NINE MILE POINT NUCLEAR STATION /P.O. ECX 32 LYCOMING, NEW YORK 1300 / TELEPHONE (315) 343-2110

November 12, 1987

Mr. William Kane Division of Reactor Projects United States Nuclear Regulatory Commission Region I 631 Park Avenue King of Prussia, PA 19406

Dear Mr. Kane:

In being consistent with our conservative philosophy, on October 13, 1987, at 1127 hours, we notified NRC Operations Center via ENS of an event which we interpreted under the amended 10CFR 73.71(c) and supporting Regulatory Guide 5.62 as a 1-hour call. However, further in depth evaluation and more significantly, guidance from our Regional Inspector, concluded that the subject incident (Licensee Occurrence Report #87=05) was in fact a 24-hour loggable event.

It was specifically requested by our Regional Inspector that we not rescind the original call via ENS rather officially submit this letter to meet our commitment.

Very truly yours,

Joseph P. Becalta

Joseph P. Beratta Manager, Nuclear Security

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