

Entergy Operations, Inc.

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February 15, 1991

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U. S. Nuclear Regulatory Commission Document Control Desk Mail Stop P1-137 Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 1

Docket No. 50-313 License No. DPR-51 Monthly Operating Report

Gentlemen:

The Arkansas Nuclear One - Unit 1 Monthly Operating Report for January, 1991 is attached. This report is submitted in accordance with ANO-1 Technical Specification 6.12.2.3. Also, included as an attachment is the "1990 Annual Report of Safety and Relief Valves Failures and Challenges" which is submitted in accordance with ANO-1 Technical Specification 6.12.2.4.

Very truly yours,

James J. Visicaro Manager, Licersing

JJF/SAB/1pi Attachment U. S. NRC February 15, 1991 Page 2

cc:

Mr. Robert D. Martin Regional Administrator U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76011

NRC Senior Resident Inspector Arkansas Nuclear One - ANO-1 & 2 Number 1, Nuclear Plant Road Russellville, AR 72801

Mr. Thomas W. Alexion NRR Project Manager, Region IV/ANO-1 U. S. Nuclear Regulatory Commission NRR Mail Stop 11-B-19 One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

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OPERATING DATA REPORT

DOCKET NO: 50-313

DATE: January, 1991

COMPLETED BY:D. A. Schaubroeck

TELEPHONE: (501 964-5535

OPERATING STATUS

Startup:

1.	Unit Name: Arkansas Nuclear One - Unit 1
2.	Reporting Period: January 1-31, 1991
3.	Licensed Thermal Power (MWt): 2,568
4.	Nameplate Rating (Gross MWe): 902,74
5.	Design Electrical Rating (Net MWe): 850
6.	Maximum Dependable Capacity (Gross MWe): 883
7.	Maximum Dependable Capacity (Net MWe): 836
8.	If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since
	Last Report, Give Reasons:
9.	Power Level To Which Restricted. If Any (Net MWe): None

9. Power Level To Which Restricted. If Any (Net MWe): None
10. Reasons For Restrictions. If Any: None

		MONTH	YR-TO-DATE	CUMULATIVE
1.	Hours in Reporting Period	744.0	744.0	141,307.0
2.	Number of Hours Reactor was Critical	459.2	459.2	98,170.6
3.	Reactor Reserve Shutdown	439.2	439.2	90,170.0
	Hours	0.0	0.0	5,044.0
4.	Hours Generator On-Line	355.3	355.3	96,094.0
5.	Unit Reserve Shutdown Hours	0.0	0.0	817.5
6.	Gross Thermal Energy Generated			
	(MWH)	783,052.0	783,052.0	215,839,169.0
7.	Gross Electrical Energy			
	Generated (MWH)	256,245.0	256,245.0	71,679,940.0
8.	Net Electrical Energy			
	Generated (MWH)	232,281.0	232,281.0	68,067,016.0
9.	Unit Service Factor	47.8	47.8	68.0
.0.	Unit Availability Factor	47.8	47.8	68.6
11.	Unit Capacity Factor			
	(Using MDC Net)	37.3	37.3	57.6
2.	Unit Capacity Factor			
	(Using DEC Net)	36.7	36.7	56.7
3.	Unit Forced Outage Rate	41.5	41.5	13.3
4.	Shutdowns Scheduled Over Next 6	Months (Type, Da	ate, and Duration	of
	Each): ANO-1 Mid-Cycle Outage S	cheduled to beg	in 4/7/91 and las	t
	approximately 3 weeks.			
5.	If Shut Down At End of Report Pe	riod. Estimated	d Date of	

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

26. Units in Test Status (Prior to Commercial Operation):

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313
UNIT: One
DATE: January, 1991
COMPLETED BY:D. A. Schaubroeck
TELEPHONE: (501) 964-5535

MONTH January, 1991

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AVGS: 312

INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY

JANUARY, 1991

UNIT ONE

Unit One began the month off line in an extension to the 1R9 refueling outage. The unit was placed on line for Cycle 10 operation on January 6 at 1204 hours.

On the sixth, at 2105 hours, the unit came off line to perform a scheduled turbine overspeed test. The unit returned to service on the seventh at 0126 hours. Power escalation to 100% was held for normal post-refueling activities such as NIS calibration, Xenon equalization, physics testing and fuel preconditioning. A brief hold also occurred due to electrical breaker problems. The unit reached 100% power on the tenth at 2208 hours.

On the tenth, at 2325 hours, the reactor tripped due to the exciter AC rotor winding fault. The unit went on line on the twenty-first at 0541 hours, and the turbine was manually tripped at 0555 hours that same day due to exciter bearing vibration.

The unit was placed back on line on the twenty-first, at 1153 hours, following bearing repairs. The unit ramped to full power with hold points for feedwater chemistry, NIS calibration and physics testing. The unit reached 100% power on the twenty-third at 1430 hours.

UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR JANUARY, 1991

								TELEPHONE	(501) 964-5535
No.	Date	Туре	Duration Hours)	Keason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
91-01	910101	S	132.1	С	4	N/A	ZZ	222222	Unit refueling outage 1R9 continued.
91-02	910106	S	4.3	В	5	N/A	ZZ	222222	Post refueling outage turbine overspeed trip test.
91-03	910110	F	246.3	A	3	1-91-001	TL	EXC	Exciter winding fault caused RX trip.
91-04	910121	F	6.0	A	5	N/A	TL	EXC	Temporary exciter bearing vibration required manual turbine trip.

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain) 1-Manual

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

2-Manual Scram.

3-Automatic Scram.

4-Continuation

5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee

Event Report (LER) File (NUREG-

50-313

January, 1991

D. A. Schaubroeck

One

DOCKET NO. UNIT NAME.

COMPLETED BY

DATE

1022)

Exhibit I - Same Source

DATE: January, 1991

REPUELING INFORMATION

1.	Name of facility: Arkansas Nuclear One - Unit 1
2.	Scheduled date for next refueling shutdown. Arril, 1992
3.	Scheduled date for restart following refueling. June. 1992
4.	Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
	Technical Specification changes will be required for ANO-1 in order to increase our fuel enrichment from 3.5% to 4.1%
5.	Scheduled date(s) for submitting proposed licensing action and supporting information. ANO is presently in the process of preparing a license amendment submittal to increase the fuel enrichment from 3.5% to 4.1%.
6.	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
	None.
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 566
8.	The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
	present 968 increase size by 0
9.	The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
	DATE: 1995 (Loss of fullcore offload capability)

ATTACHMENT

AUNUAL REPORT OF SAFETY VALVE

AND RELIEF VALVE

FAILURES AND CHALLENGES

This annual report is submitted in the January Monthly Operating Peport in response to requirements implemented as a result of NUREG-0737, Item II.K.3.3 and to fulfill Technical Specification reporting requirements (TS 6.12.2.4 for Unit 1 and TS 6.9.1.5.C for Unit 2).

ANO-1: no challenges to the primary system code safeties nor electromatic relief valve (ERV) have occurred in the year 1990.

ANO-2: no challenges to the primary system code safeties have occurred in the year 1990. ANO-2 does not have an ERV.