



Callaway Plant

January 25, 1991

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-2358

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 90-017-00
A REACTOR TRIP DUE TO A FAILURE OF A CONTROLLER/DRIVER
CARD FOR THE 'B' FEEDWATER REGULATING VALVE

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning a reactor trip due to a failure of a controller/driver card for the 'B' Feedwater Regulating Valve.

J. D. Blosser
J. D. Blosser
for Manager, Callaway Plant

JDB/TPS/MAH/lrj

Enclosure

cc: Distribution attached

9101280180 910125
PDR ADOCK 05000483
S PDR

JE22
11

cc distribution for ULNRC-2358

Mr. A. Bert Davis
Regional Administrator
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road Glen Ellyn, IL 60137

Manager, Electric Department
Missouri Public Service Commission
P.O. Box 360
Jefferson City, MO 65102

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

NRC Senior Resident Inspector

Mr. M. David Lynch (2 copies)
U.S. Nuclear Regulatory Commission
OWPN - Mail Stop 13E21
Washington, D.C. 20555

Mr. O. Maynard
Wolf Creek Nuclear Operating Corp.
P. O. Box 411
Burlington, KS 66839

Mr. Merlin Williams
Supt. of Regulatory Quality
& Administrative Services
Wolf Creek Nuclear Operating Corp.
P. O. Box 411
Burlington, KS 66839

Mr. R. L. Hague
Chief, Project Section 3C
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Callaway Plant Unit 1						DOCKET NUMBER (2) 0 5 0 0 0 4 8 3			PAGE (3) 1 OF 0 3		
--	--	--	--	--	--	--------------------------------------	--	--	----------------------	--	--

TITLE (4)
A Reactor Trip Due To A Failure Of A Controller/Driver Card For The 'B' Feedwater Regulating Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)															
1	2	3	0	9	0	9	0	0	1	7	0	0	0	0	1	2	5	9	1			0	5	0	0	0

OPERATING MODE (8)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)										
POWER LEVEL (10) 1 0 0	1	20.402(b)		20.405(c)	X	50.73(a)(2)(iv)		73.71(b)				
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)				
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract Below and in Text, NRC Form 366A)				
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)									
NAME K. R. Evans, Instrument & Control							TELEPHONE NUMBER 3 1 4 6 7 6 - 8 6 4 5		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	
X	S	J C A P	W	3 5 1						

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		
YES (If yes, complete EXPECTED SUBMISSION DATE)						NO		
						MONTH DAY YEAR		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 12/30/90, at 1152 CST, a reactor trip occurred due to the failure of a controller/driver card for the 'B' Feedwater Regulating Valve (FRV). The FRV failed closed and could not be re-opened by the licensed operators from the Main Control Board in either the automatic or manual mode. The subsequent 'B' Steam Generator low water level actuated the reactor trip signal. As a result of the RPS actuation, a feedwater isolation (FWIS) and an auxiliary feedwater actuation (AFAS) were generated by design. The plant was in Mode 1 - Power Operations at 100 percent reactor power. The reactor coolant system temperature was 588 degrees F and the pressure was 2235 psig. The licensed operators recovered from the trip and Engineered Safety Feature (ESF) actuations via plant procedures.

The FRV controller/driver card was replaced at 1330 on 12/30/90. The manufacturer of the card is Westinghouse (component #2837A16G03). The plant was returned to Mode 1 - Power Operations at 0134 on 12/31/90. A capacitor in the power supply section of the card was the cause of the card failure. The failure of the capacitor is indeterminate. Corrective actions include the failure history and other applications of the use of this capacitor will be evaluated; and an evaluation will be performed to determine the feasibility of adding a redundant controller/driver card to the circuit for the FRV's.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-830), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 6 0 0 0 4 8 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	- 0 1 7	- 0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 305A's) (17)

Basis for Reportability

On 12/30/90 at 1152 CST, an automatic reactor protection system (RPS) reactor trip occurred on steam generator low level. As a result of the RPS actuation, a feedwater isolation (FWIS) and an auxiliary feedwater actuation (AFAS) were generated by design. These ESF actuations are reportable under CFR.73(a)(2)(iv).

Plant Conditions at Time of Event

Mode 1 - Power Operations 100 percent Reactor Power.
Reactor Coolant System (RCS) Temperature (ave) - 588 degrees F.
RCS Pressure - 2235 psig.

Description of Event

On 12/30/90, at 1152 CST, a reactor trip occurred due to the failure of a controller/driver card for the 'B' Feedwater Regulating Valve (FRV). The FRV failed closed and could not be re-opened by the licensed operators from the Main Control Board in either the automatic or manual mode. The subsequent 'B' Steam Generator low water level actuated the reactor trip signal. The operators had decided to initiate a manual trip, but the automatic trip signal actuated first, approximately 24 seconds after the initial indications of the component failure. The licensed operators recovered from the trip and ESF actuations via plant procedures. The FRV controller/driver card was replaced by 1330 on 12/30/90. The plant was returned to Mode 1 - Power Operations at 0134 on 12/31/90.

Root Cause

This event was caused by the failure of the controller/driver card for the 'B' FRV. The manufacturer of the card is Westinghouse (component #2837A16G03). A 2.2 μ F, 50 volt ceramic monolithic capacitor in the power supply section of the card was the cause of the card failure. The failure of the capacitor is indeterminate.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-330), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 9 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 0	0 1 7	0 0	0 3	OF	0 3

TEXT (if more space is required, use additional NRC Form 366A's) (17)

Corrective Actions

1. The FRV controller/driver card was replaced and tested satisfactorily. The failed component in the card has been determined to be a capacitor in the power supply section of the card. The failure history and other applications of the use of this capacitor will be evaluated.
2. Evaluation will be performed to determine feasibility of adding a redundant controller driver card to the circuit for the FRV's.

Safety Significance

Plant equipment functioned as required by design. There was no detrimental effect on plant equipment. The safety related equipment performed as expected and no unusual radiation readings or chemistry results were attributed to the event. The event did not pose a threat to the health and safety of the public.

Previous Occurrences

None

Footnotes

The system and component codes listed below are from IEEE Standards 805-1984 and 803A-1983, respectively.

- 1) System - SJ, Components - CBD, 20, and CAP
- 2) System - JC
- 3) System - AB
- 4) System - JE