

## NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20656

February 12, 1991

Docket Nos. 50-317 and 50-318

Mr. G. C. Creel
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
MD Rts. 2 & 4
P. O. Box 1535
Lusby, Maryland 20657

Dear Mr. Creel:

SUBJECT: RESPONSE TO STATION BLACKOUT RULE - CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2, TAC NUMBERS 68525 (UNIT 1) AND 68526 (UNIT 2)

On July 21, 1988, the Code of Federal Regulations 10 CFR Part 50, was amended to include a new Section 50.63, entitled, "Loss of All Alternating Current Power," (Station Blackout). The station blackout (SBO) rule requires that each light-water-cooled nuclear power plant be able to withstand and recover from an SBO of specified duration. The SBO rule also requires that information defined in the rule be provided to the staff for review.

By letters dated April 14, 1989, March 30 and September 24, 1990, you provided your response to the SBO rule for the Calvert Cliffs Nuclear Power Plant, Units 1 and 2. The NRC staff's Safety Evaluation (SE) pertaining to your initial responses was transmitted to you by letter dated October 10, 1990. The staff found the proposed method of coping with an SBO to be acceptable, subject to the satisfactory resolution of several recommendations which were itemized in the staff's SE. You responded to the staff's SE, and specifically to the recommendations, by letters dated November 13, 1990 and December 13, 1990.

The staff has reviewed your responses to its SE and determined them acceptable as detailed in the enclosed Supplemental Safety Evaluation (SSE) the staff has stated some "understandings" in the SSE which should be reviewed and responded to if your understanding is different than the staffs. These understandings are that: (1) the modifications associated with the existing emergency diesel generator (EDG) that is proposed as the Alternate AC (AAC) source (as well as the new EDGs) will be submitted for staff review; (2) that the documentation associated with the testing of the AAC source will be maintained with the other SBO documentation; and (3) for the calculated temperature values expected in the nine areas containing SBO equipment, that their is reasonable assurance of the operability of SBO response equipment based on your assessment. It is noted that Technical Specification changes associated with the overall project will be submitted for staff review. It is

9102200026 910212 PDR ADDCK 05000317 FDR PDR also noted that if the resolution of Generic Issue 23 defines a higher reactor coolant pump leakage than assumed in your supporting analysis, the impact will have to be addressed.

This completes our action relative to the above referenced TAC numbers.

Sincerely,

Original Signed By:

Daniel G. McDonald, Senior Project Manager Project Directorate I-1 Division of Reactor Project - I/II Office of Nuclear Reactor Regulation

Enclosure: As stated

cc: w/enclosure See next page

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