

February 15, 1991

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U. S. Nuclear Regulatory Commission Document Control Desk Mail Stop P1-137 Washington, D.C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2

Docket No. 50-368 License No. NPF-6

Monthly Operating Report

#### Gentlemen:

The Arkansas Nuclear One - Unit 2 Monthly Operating Report for January, 1991 is attached. This report is submitted in accordance with ANO-2 Technical Specification 6.9.1.6. Also, included as an attachment is the "1990 Annual Report of Safety and Relief Valves Failures and Challenges" which is submitted in accordance with ANO-2 Technical Specification 6.9.1.5(c).

Very truly yours,

James J. Fisicaro Manager, Licensing

JJF/SAB/1pi Attachment

CC:

Mr. Robert D. Martin Regional Administrator U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76011

NRC Senior Resident Inspector Arkansas Nuclear One - ANO-1 & 2 Number 1, Nuclear Plant Road Russellville, AR 72801

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Ms. Sheri Peterson NRR Project Manager, Region IV/ANO-2 U. S. Nuclear Regulatory Commission NRR Mail Stop 11-B-19 One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

#### OPERATING DATA REPORT

DOCKET NO: 50-368 January, 1991 COMPLETED BY: M. S. Whitt

TELEPHONE: (501 964-5560

# OPERATING STATUS

Unit Name: Arkansas Nuclear One - Unit 2 Reporting Period: January 1-31, 1991 2. Licensed Thermal Power (MWt): 2,815 3. 4. Nameplate Rating (Gross MWe): 942.57 Design Electrical Rating (Net MWe): 912 5. Maximum Dependable Capacity (Gross MWe): 897 6. Maximum Dependable Capacity (Net MWe): 858 7.

If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since 8. Last Report, Give Reasons:

Power Level To Which Restricted. If Any (Net MWe): None 10. Reasons For Restrictions. If Any: None

A A SECURITY				
		MONTH	YR-TO-DATE	CUMULATIVE
11.	Hours in Reporting Period	744.0	744.0	95,136.0
12.	Number of Hours Reactor was Critical	744.0	744.0	71,354.7
13.	Reactor Reserve Shutdown	7.541.0	7.441.0	71,00417
	Kours	0.0	0.0	1,430.1
14.	Hours Generator On-Line	744.0	744.0	69,737.8
15.	Unit Reserve Shutdown Hours	0.0	0.0	75.0
16.	Gross Thermal Energy Generated			
	(MWH)	1,993,107.0	1,993,107.0	182,811,948.0
17.	Gross Electrical Energy			
	Generated (MWH)	670,180.0	670,180.0	60,099,426.0
18.	Net Electrical Energy			
	Generated (MWH)	640,983.0	640,983.0	57,147,521.0
19.	Unit Service Factor	100.0	100.0	73.3
20.	Unit Availability Factor	100.0	100.0	73.4
21.	Unit Capacity Factor			
	(Using MDC Net)	100.4	100.4	70.0
22.	Unit Capacity Factor			
	(Using DEC Net)	94.5	94.5	65.9
23.	Unit Forced Outage Rate		0.0	12.6
24.	Shutdowns Scheduled Over Next 6			
	Each): 2R8 Refueling Outage is			li
	and the scheduled date for rest	(		
25.	If Shut Down At End of Report P	eriod. Estimate	d Date of	
	At a second seco			

	Forecast	Achieved
INITIAL CRITICALITY	Annual Contraction	*****
INITIAL ELECTRICITY COMMERCIAL OPERATION		-
COURTRAL OLDERATION	AND DESCRIPTION OF THE PARTY.	-

26. Units in Test Status (Prior to Commercial Operation):

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368
UNIT: Two
DATE: January, 1991
COMPLETED RY: M. S. Whitt
TELEPHONE: (501) 964-5560

# MONTH January, 1991

DAY	AVERAGE			LEVEL
	()	We-Net	)	
1	222222	903		
2		902		
3	315 317 7 7 7	902		
4	*****	902		
5		902		
6		902		
7		903		
8	*****	902		
9	*****	902		
10		901		
11		902		
12		902		
13		902		
14		901		
15		901		
16		901		
17		900		
18		893		
19		856		
20		818		
21		803		
22		802		
23		801		
24		798		
25		791		
26		778		
27		815		
28		817		
29		802		
**		WV.K		

AVGS: 862

30 ..... 805

# INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

#### NRC MONTHLY OPERATING REPORT

#### OPERATING SUMMARY

JANUARY, 1991

UNIT TWO

The unit began the month operating at 100% full power (FF).

On January eighteenth, at 1415 hours, a power reduction to 97.5% was commenced due to a Control Element Assembly (CEA) dropping into the Reactor core. After the CEA was withdrawn, power was further reduced to 88% per the dispatcher's request on the same day at 2200 hours. The reduction in power to 88% was required to minimize the coastdown time at the end of the cycle because of fuel limitations. On the twenty-fifth, at 1846 hours, power was reduced to 86% in order to locate and plug a leaking condenser tube.

On the twenty-sixth, at 2237 hours, power was returned 38% and the unit operated on that level through end of the month.

# UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR JANUARY, 1991

DOCKET NO. 50-368 UNIT NAME Two Jacuary, 1991 DATE COMPLETED BY M. S. Whitt TELEPHONE (501) 964-5560

Method of Licensee Cause & Corrective Duration Shutting Event System Component Action to No. Date Type1 (Hours) Down Reactor3 Code<sup>5</sup> Reason<sup>2</sup> Report # Code\* Prevent Recurrence

None

F: Forced S: Scheduled

Reason:

A-Equipment Failure (Explain) 1-Manual

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

2-Manual Scram.

3-Automatic Scram.

4-Continuation

5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-10223

Exhibit I - Same Source

DATE: January, 1991

# REFUELING INFORMATION

1.	Name of facility: Arkansas Nuclear One - Unit 2				
2.	Scheduled date for next refueling shutdown. February, 1991 (Beginning of Cycle 8 criticality was 11/18/89).				
3.	Scheduled date for restart following refueling. April, 1991				
4.	Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?				
	None Expected. Reload fuel design is in progress.				
5.	Scheduled date(s) for submitting proposed licensing action and supporting information. None Required.				
6,	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.				
	To obtain the presently planned cycle 8 length of 420 EFPD, it will to necessary to raise the current peak rod burnup limits. A report justifying an increase was submitted in July, 1989*				
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, a) 177 b) 489				
8.	The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.				
	present 968 increase size by 0				
9.	The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.				
	DATE: 1996 (Loss of fullcore offload capability)				
	*NOTE: NRC approval was given on the Extended Burnup Topical in November, 1990. Therefore, allowing Cycle 8 operation to continue at its specified cycle length.				

### ATTACHMENT

#### ANNUAL REPORT OF SAFETY VALVE

#### AND RELIEF VALVE

# FAILURES AND CHALLENGES

This annual report is submitted in the January Monthly Operating Report in response to requirements implemented as a result of NUREG-0737, Item II.K.3.3 and to fulfill Technical Specification reporting requirements (TS 6.12.2.4 for Unit 1 and TS 6.9.1.5.0 for Unit 2).

ANO-1: no challenges to the primary system code safeties nor electromatic relief valve (ERV) have occurred in the year 1990.

ANO-2: no challenges to the primary system code safeties have occurred in the year 1990. ANO-2 does not have an ERV.