



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated July 30, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

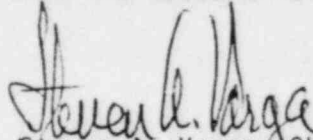
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 10, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-66

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Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-34	3/4 3-34
-	3/4 3-34a
3/4 3-35	3/4 3-35
3/4 3-36	3/4 3-36
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B 3/4 3-2	B 3/4 3-2

Revise Appendix B as follows:

Table 2.4-4

Table 2.4-4

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT#</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
1. AREA MONITORS					
a. Fuel Storage Pool Area (RM-207)	1	*	≤15 mR/hr	10 ⁻¹ - 10 ⁴ mR/hr	19
b. Containment					
i. Purge & Exhaust Isolation (RMVS 104 A & B)	1	6	≤1.6 x 10 ³ cpm	10 - 10 ⁶ cpm	22
ii. Area (RM-RM-219 A & B)	2	1, 2, 3, & 4	≤30 R/hr	1 - 10 ⁷ R/hr	36
2. PROCESS MONITORS					
a. Containment					
i. Caseous Activity					
Purge & Exhaust Isolation (RM-215B)	1	6	≤7.3 x 10 ² cpm	10 - 10 ⁶ cpm	22
RCS Leakage Detection (RM 215B)	1	1, 2, 3, & 4	N/A	10 - 10 ⁶ cpm	20
ii. Particulate Activity					
Purge & Exhaust Isolation (RM 215A)	1	6	≤2.5 x 10 ⁵ cpm	10 - 10 ⁶ cpm	22
RCS Leakage Detection (RM 215A)	1	1, 2, 3, & 4	N/A	10 - 10 ⁶ cpm	20
b. Fuel Storage Building Gross Activity (RMVS - 103 A & B)	1	**	≤4.0 x 10 ⁴ cpm	10 - 10 ⁶ cpm	21

* With fuel in the storage pool or building

* With irradiated fuel in the storage pool

Above background

TABLE 3.3-6 (Continued)

RADIATION MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>SETPOINT#</u>	<u>MEASUREMENT RANGE</u>	<u>ACTION</u>
PROCESS MONITORS (Continued)					
c. Noble Gas Effluent Monitors					
i. Supplementary Leak Collection and Release System (RM-VS-110 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 3.5 \times 10^2$ cpm	$10^{-2} - 10^5$ uCi/cc*	36
ii. Auxiliary Building Ventilation System (RM-VS-109 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 2.75 \times 10^2$ cpm	$10^{-2} - 10^5$ uCi/cc*	36
iii. Process Vent System (RM-GW-109 Ch. 7 & Ch. 9) ***	1	1, 2, 3, & 4	$\leq 1.8 \times 10^4$ cpm	$10^{-2} - 10^5$ uCi/cc**	36
iv. Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge (RM-MS-100 A, B, C)	1/S.G	1, 2, 3, & 4	$\leq 5.0 \times 10^1$ cpm	$10^{-1} - 10^3$ uCi/cc	36
v. Auxiliary Feedwater Pump Turbine Exhaust (RM-MS-101)	1	1, 2, 3, & 4	$\leq 6.5 \times 10^2$ cpm	$10^{-1} - 10^3$ uCi/cc	36

* Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 7

** Nominal range for Ch. 7 and Ch. 9. Alarm set on Ch. 9

*** Other SPINC-4 channels not applicable to this specification

Above background

TABLE 3.3-6 (Continued)

TABLE NOTATION

- ACTION 19 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 20 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 21 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the applicable ACTION requirements of Specifications 3.9.12 and 3.9.13.
- ACTION 22 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 36 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable Channel(s) to OPERABLE status within 72 hours,
or:
- 1) Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - 2) Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. AREA MONITORS				
a. Fuel Storage Pool Area (RM 207)	S	R	M	*
b. Containment				
i. Purge & Exhaust Isolation (RMVS 104 A & B)	S	R	M	6
ii. Area (RM-RM-219 A & B)	S	R	M	1, 2, 3, & 4
2. PROCESS MONITORS				
a. Containment				
i. Gaseous Activity				
Purge & Exhaust Isolation (RM 215B)	S	R	M	6
RCS Leakage Detection (RM 215B)	S	R	M	1, 2, 3, & 4
ii. Particulate Activity				
Purge & Exhaust Isolation (RM 215A)	S	R	M	6
RCS Leakage Detection (RM 215A)	S	R	M	1, 2, 3, & 4
b. Fuel Storage Building Gross Activity (RMVS 103 A & B)	S	R	M	**

* With fuel in the storage pool or building
 ** With irradiated fuel in the storage pool

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TABLE 4.3-3 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE IS REQUIRED</u>
PROCESS MONITORS (Continued)				
c. Noble Gas Effluent Monitors				
i. Supplementary Leak Collection and Release System (RM-VS-110 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
ii. Auxiliary Building Ventilation System (RM-VS-109 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
iii. Process Vent System (RM-GW-109 Ch. 7 & Ch. 9)	S	R	M	1, 2, 3, & 4
iv. Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge (RM-MS-100 A, B, C)	S	R	M	1, 2, 3, & 4
v. Auxiliary Feedwater Pump Turbine Exhaust (RM-MS-101)	S	R	M	1, 2, 3, & 4

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AMENDMENT NO. 59

INSTRUMENTATION

BASES

3/4.3.3 MONITORING INSTRUMENTATION

3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that:

- 1) the radiation levels are continually measured in the areas served by the individual channels; 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded; and 3) sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of NUREG-0737, "Clarification of TMI Action Plan Requirements," October, 1980.

3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the reactor core. The OPERABILITY of this system is demonstrated by irradiating each detector used and determining the acceptability of its voltage curve.

For the purpose of measuring $F_0(z)$ or $F_{\Delta H}^N$, a full incore flux map is used. Quarter-core flux maps, as defined in WCAP-8648, June 1976, may be used in recalibration of the excore neutron flux detection system, and full incore flux maps or symmetric incore thimbles may be used for monitoring the Quadrant Power Tilt Ratio when one Power Range Channel is inoperable.

3/4.3.3.3 SEISMIC INSTRUMENTATION

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the facility and is consistent with the recommendations of Regulatory Guide 1.12, "Instrumentation for Earthquakes."

3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public and is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs."

TABLE 2.4-4

PRESSURIZED WATER REACTOR GASEOUS WASTE SYSTEM
LOCATION OF PROCESS AND EFFLUENT MONITORS AND SAMPLERS REQUIRED BY TECHNICAL SPECIFICATIONS

Process Stream or Release Point	Alarm ^b	Auto Control to Isolation Valve or Filters	Continuous Monitor	Grab Sample Station	Measurement				
					Noble Gas	I	Particulate	H-3	Alpha
Waste Gas Storage Tanks ⁽²⁾ RM-GW-101	X	X	X	X	X	X	X	X	X
Condenser Air Ejector ^{a(2)} RM-SV-100	X	X	X		X	X	X	X	X
Vent Header System ^{a(2)}	X		X	X	X	X	X	X	X
Atmospheric Steam Dump Valve and Code Safety Relief Valve Discharge			X		X				
Auxiliary Feedwater Pump Turbine Exhaust			X		X				
<u>Building Ventilation Systems</u>									
Reactor Containment Building (whenever there is flow) ^{a(1)} or (3) RM-VS-105	X	X	X	X	X	X	X	X	X
Auxiliary Building ^{a(3)} or (1) RM-VS-102 A & B	X		X	X	X	X	X	X	X
Lab Hoods and PCA Shop ^{a(3)}	X		X	X	X	X	X	X	X
<u>Supplementary Leak Collection System</u>									
Fuel Building ^{a(1)} RM-VS-103 A & B	X		X	X	X	X	X	X	X
Containment Vacuum Pumps ^{a(2)}	X		X	X	X	X	X	X	X

^aIf any or all of the process streams or building ventilation systems are routed to a single release point, the need for a continuous monitor at the individual discharge point to the main exhaust duct is eliminated. One continuous monitor and measurement capability at the final release point is sufficient.

- (1) Elevated release - Top Reactor Containment. RM-VS-107 A & B
 (2) Process Vent - Top Cooling Tower. RM-GW-108 A & B
 (3) Ventilation Vent - Roof Aux. Bldg. RM - VS-101 A & B

^bAlarms are at downstream locations, not necessarily specific to the stated process stream, but to its final release point [footnote a(1), a(2) and a(3)]