## ATTACHMENT B

6

# PROPOSED CHANGES TO THE TECHNICAL SPECIFICATIONS FOR OPERATING LICENSES NPF-11 AND NPF-18

# REVISED PAGES

# UNIT 1 (NPF-11) UNIT 2 (NPF-18) 3/4 3-11 3/4 3-11 3/4 3-14 (a) 3/4 3 14 (a) B 3/4 3-2 B 3/4 3-2

9101240353 910118 PDR ADOCK 05000373 PDR

-

 $\mathcal{O}_{ij} = \{i_1, j_2, \dots, j_{i_j}\}$ 

## TABLE 3.3.2-1

# ISOLATION ACTUATION INSTRUMENTATION

2

TRI	IP FU	NCTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (D)	APPLICABLE OPERATIONAL CONDITION	ACTION
Α.	AUTOMATIC INITIATION					
1.	PRI	IMARY CONTAINMENT ISOLATION				
	a.	Reactor Vessel Water Level (1) '.ow, Level 3 (2) Low Low, Level 2 (3) Low Low Low, Level 1	7 2, 3 1, 10	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20
	b.	Drywell Pressure - High	2, 7, 10	2	1, 2, 3	20
	с.	Main Steam Line 1) Radiation - High 2) Pressure - Low 3) Flow - High	] 3 ]	2 2 2/line(d)	1, 2, 3 1, 2, 3 1	21 22 23
	d.	Main Steam Line Tunnel Temperature - High	1	2	1. 2. 3 4	zi (i)(j)
	e.	Main Steam Line Tunnel ∆Temperature - High	l	2	1 <sup>(i)</sup> , 2 <sup>(i)</sup> , 3 <sup>(i)</sup>	1º21 (1)
	f.	Condenser Vacuum - Low	1	2	1, 2*, 3*	21 (01)
2.	SECO	ONDARY CONTAINMENT ISOLATION				00
	a.	Reactor Building Vent Exhaust Plenum Radiation - High	4(c)(e)	2	1, 2, 3 and **	24
	b.	Drywell Pressure - High	4(c)(e)	2	1, 2, 3	24
	с.	Reactor Vessel Water Level - Low Low, Level 2	4(c)(e)	2	1, 2, 3, and #	24
	đ.	Fuel Pool Vent Exhaust Radiation - High	4(c)(e)	2	1, 2, 3, and **	24

3/4 3-11

の記録

10

Amendment No.50

÷.,

## TABLE 3.3.2-1 (Continued)

## NOTES (Continued)

- (g) Requires RCIC steam supply pressure-low coincident with drywell pressure-high.
- (h) Manual initiation isolates 1E51-FDOB only and only with a coincident reactor vessel water level-low, level 2, signal.
   (i) Both channels of each trip system may be placed in an inoperable status for up
- to 4 hours for required reactor building ventilation filter change and damper cycling without placing the trip system in the tripped condition provided that the ambient temperature channels in the same trip systems are operable.



1

•

1

## INSTRUMENTATION

### BASES

# 3/4.3.2 ISOLATION ACTUATION INSTRUMENTATION

This specification ensures the effectiveness of the instrumentation used to mitigate the consequences of accidents by prescribing the OPERABILITY trip setpoints and response times for isolation of the reactor systems. When necessary, one channel may be inoperable for brief intervals to conduct required surveillance. Some of the trip settings may have tolerances explicitly stated where both the high and low values are critical and may have a substantial effect on safety. The setpoints of other instrumentation, where only the high or low end of the setting have a direct bearing on safety, are established at a level away from the normal operating range to prevent inadvertant actuation of the systems involved.

14

Except for the MSIVs, the safety analysis does not address individual sensor response times or the response times of the logic systems to which the sensors are connected. For D.C. operated valves, a 3 second delay is assumed before the valve starts to move. For A.C. operated valves, it is assumed that the A.C. power supply is lost and is restored by startup of the emergency diesel generators. In this event, a time of 13 seconds is assumed before the valve starts to move. In addition to the pipe break, the failure of the D.C. operated valve is assumed; thus the signal delay is concurrent with the 13 second diesel startup. The safety analysis considers an allowable inventory loss in each case which in turn determines the valve speed in conjunction with the 13 second delay. It follows that checking the valve speeds and the 13 second time for emergency However, to enhance overall system reliability and to monitor instrument channel response time trends, the isolation actuation instrumentation response time shall be measured and recorded as a part of the ISOLATION SYSTEM RESPONSE TIME.

# 3/4.3.3 EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

The emergency core cooling system actuation instrumentation is provided to initiate actions to mitigate the consequences of accidents that are beyond the ability of the operator to control. This specification provides the OPERABILITY requirements, trip setpoints and response times that will ensure effectiveness of the systems to provide the design protection. Although the instruments are listed by system, in some cases the same instrument may be used to send the actuation signal to more than one system at the same time.

CON N

8 3/4 3-2

# TABLE 3.3.2-1

# ISOLATION ACTUATION INSTRUMENTATION

TRI	P FU	ACTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (b)	APPLICABLE OPERATIONAL CONDITION	ACTION			
Α.	AUT	TOMATIC INITIATION							
1.	PRI	MARY CONTAINMENT ISOLATION							
	a.	Reactor Vessel Water Level (1) Low, Level 3 (2) Low Low, Level 2 (3) Low Low Low, Level 1	7 2, 3 1, 10	2 2 2	1, 2, 3 1, 2, 3 1, 2, 3	20 20 20			
	ь.	Drywell Pressure - High	2, 7, 10	2	1. 2. 3	20			
	ς.	Main Steam Line 1) Radiation - High	1 3	2 2	1, 2, 3 1, 2, 3	21 22			
		2) Pressure - Low 3) Flow - High	1	2 2/line <sup>(d)</sup>	1 1, 2, 3	23			
	d.	Main Steam Line Tunnel Temperature - High	1	2	1 2 3 3	21 (1) (			
	e,	Main Steam Line Tunnel A Temperature - High	1	2	1(i), 2(i), 3(i	1/21			
	f.	Condenser Vacuum - Low	1	2	1, 2*, 3*	21 2:			
2.	SEC	SECONDARY CONTAINMENT ISOLATION							
	a.	Reactor Building Vent Exhaust Plenum Radiation - High	4(c)(e)	2	1, 2, 3 and **	24			
	b.	Drywell Pressure - High	4(c)(e)	2	1, 2, 3	24			
	С.	Reactor Vessel Water Level - Low Low, Level 2	4(c)(e)	2	1, 2, 3, and #	24			
	d.	Fuel Pool Vent Exhaust Radiation - High	4(c)(e)	2	1, 2, 3, and **	24			

LA SALLE - UNIT 2

3/4 3-11

(i) Both channels of each trip system may be placed in an inoperable status for up to 4 hours for required reactor building ventilation filter change and damper cycling without placing the trip system in the tripped condition provided that the ambient temperature channels in the sime trip systems are OPERABLE.

REPLACE WITH INSERT A

9

#### INSTRUMENTATION

BASES

#### 3/4.3.2 ISOLATION ACTUATION INSTRUMENTATION

INSERT

This specification ensures the effectiveness of the instrumentation used to mitigate the consequences of accidents by prescribing the OPERABILITY trip setpoints and response times for isolation of the reactor systems. When necessary, one channel may be inoperable for brief intervals to conduct required surveillance. Some of the trip settings may have tolerances explicitly stated where both the high and low values are critical and may have a substantial effect on safety. The setpoints of other instrumentation, where only the high or low end of the setting have a direct bearing on safety, are established at a level away from the normal operating range to prevent inadvertent actuation of the systems involved.

Except for the MSIVs, the safety analysis does not address individual sensor response times or the response times of the logic systems to which the sensors are connected. For D.C. operated valves, a 3 second delay is assumed before the valve starts to move. For A.C. operated valves, it is assumed that the A.C. power supply is lost and is restored by startup of the emergency diesel generators. In this event, a time of 13 seconds is assumed before the valve starts to move. In addition to the pipe break, the failure of the D.C. operated valve is assumed; thus the signal delay is concurrent with the 13 second diesel startup. The safety analysis considers an allowable inventory loss in each case which in turn determines the valve speed in conjunction with the 13 second delay. It follows that checking the valve speeds and the 13 second time for emergency power establishment will establish the response time for the isolation functions. However, to enhance overall system reliability and to monitor instrument channel response time trends, the isolation actuation instrumentation response time shall be measured and recorded as a part of the ISOLATION SYSTEM RESPONSE TIME.

## 3/4.3.3 EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

The emergency core cooling system actuation instrumentation is provided to initiate actions to mitigate the consequences of accidents that are beyond the ability of the operator to control. This specification provides the OPERABILITY requirements, trip setpoints and response times that will ensure effectiveness of the systems to provide the design protection. Although the instruments are listed by system, in some cases the same instrument may be used to send the actuation signal to more than one system at the same time.

### INSERT A

- Both channels of each trip system may be placed in an inoperable status for up to 4 hours for required reactor building ventilation system corrective maintenance, filter changes, damper cycling and surveillance tests, other than Surveillance Requirement 4.6.5.1.c, without placing the trip system in the tripped condition..
- Both channels of each trip system may be placed in an inoperable status for up to 12 hours for performance of Surveillance Requirement 4.6.5.1.c without placing the trip system in the tripped condition.

### INSERT B

Both channels of each trip system for the main steam tunnel ambient temperature and ventilation system differential temperature may be placed in an incperable status for up to 4 hours for required reactor building ventilation system maintenance and testing and 12 hours for the required secondary containment Leak Rate test without placing the trip system in the tripped condition. This will allow for maintaining the reliability of the ventilation system and secondary containment.

ZNLD654/15

L.

J.