

OPERATING DATA REPORT

BUCKET NO. 50-321
 DATE 11-10-82
 COMPLETED BY HARRON SMITH
 TELEPHONE (912) 367-7781 x 203

OPERATING STATUS

* Notes *
 * * * * *

- 1. Unit Name: E I Hatch Nuclear Plant Unit 1
- 2. Reporting Period: 10-82
- 3. Licensed Thermal Power (MWt): 2436
- 4. Nameplate Rating (Gross MWe): 809.3
- 5. Design Electrical Rating (Net MWe): 777.3
- 6. Maximum Dependable Capacity (Gross MWe): 801.2
- 7. Maximum Dependable Capacity (Net MWe): 761.7
- 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

- 9. Power Level To Which Restricted, If Any (Net MWe):
- 10. Reasons For Restrictions, If Any:

This Month Yr.-to-Date Cumulative

11. Hours In Reporting Period	744	7285	59963
12. Number Of Hours Reactor Was Critical	211.2	4575.6	42966.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	212.1	4332.1	40166.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	467328	9470911	84248363
17. Gross Electrical Energy Generated (MWH)	141460	3048760	27263439
18. Net Electrical Energy Generated (MWH)	130806	2884287	25093065
19. Unit Service Factor	28.5	59.4	67.1
20. Unit Availability Factor	28.5	59.4	67.1
21. Unit Capacity Factor (Using MDC Net)	23.1	51.9	56.7
22. Unit Capacity Factor (Using DER Net)	22.6	50.9	55.6
23. Unit Forced Outage Rate	0.0	26.0	20.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation): Forecast Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

(9/27)

821190257 821115
 PDR ADDCK 03000321
 PDR
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 59-321
 DATE 11-10-82
 COMPLETED BY BARON SMITH
 TELEPHONE (912) 367-7761 x. 263

MONTH 10-82

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	710	17	-7
2	701	18	-6
3	704	19	-7
4	705	20	-7
5	703	21	-6
6	701	22	-6
7	699	23	-6
8	664	24	-6
9	22	25	-6
10	-17	26	-6
11	-13	27	-5
12	-9	28	-5
13	-9	29	-5
14	-9	30	-5
15	-8	31	-5
16	-7		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 11-8-82
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851 X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-63	821002	F	2.2	A	5	NA	CB	MOTORX	Loading reduction to repair "A" recirc MG-set brushes.
82-64	821008	F	0.5	A	5	NA	IA	PUMPXX	"B" recirc pump trip.
82-65	821008	S	654.0	C	1	NA	RC	FUELXX	Normal reactor shutdown for refueling outage.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

NARRATIVE REPORT
UNIT 1

October 2nd Load reduction at 2000 hours to repair "A"
 recirc MG-set brushes.

October 7th "B" recirc pump tripped at 1205 hours.

October 8th Normal reactor shutdown for refueling at
 2200 hours.

OPERATING DATA REPORT

COCKET NO. 50-266
 DATE 11-10-82
 COMPLETED BY: BARRY SMITH
 TELEPHONE (912) 367-7791 X. 203

OPERATING STATUS

 * Notes
 *
 1. Unit Name: E-1 Hatch Nuclear Plant, Unit 2 *
 2. Reporting Period: 10-82 *
 3. Licensed Thermal Power (MW): 2436 *
 4. Nonplate Rating (Gross MWe): 817.0 *
 5. Design Electrical Rating (Net MWe): 784.0 *
 6. Maximum Dependable Capacity (Gross MWe): 803.9 *
 7. Maximum Dependable Capacity (Net MWe): 770.9 *
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last P. Port., Give Reasons: *****

9. Power Level To Which Restricted, If Any (Net MWe):
 10. Reasons For Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	7293	27672
12. Number Of Hours Reactor Was Critical	725.3	4446.7	19744.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	714.7	4171.7	18735.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWh)	1505326	8045827	40375451
17. Gross Electrical Energy Generated (MWh)	527240	2821860	13194370
18. Net Electrical Energy Generated (MWh)	514041	2672029	12522289
19. Unit Service Factor	96.1	57.2	67.7
20. Unit Availability Factor	96.1	57.2	67.7
21. Unit Capacity Factor (Using NEC Net)	89.8	47.5	58.8
22. Unit Capacity Factor (Using DER Net)	88.1	46.7	57.9
23. Unit Forced Outage Rate	3.9	2.3	13.2
24. Outages Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CECILITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DUCKET NO 50-766
DATE 11-10-52
COMPLETED BY ARSON SMITH
TELEPHONE (404) 367-7761 X 203

NORTH 16-52

DAY	AVERAGE DAILY POWER LEVEL (MW-Net)	DAY	AVERAGE DAILY POWER LEVEL (MW-Net)
1	747	17	753
2	732	18	784
3	655	19	762
4	771	20	781
5	775	21	778
6	775	22	760
7	714	23	772
8	736	24	764
9	614	25	789
10	752	26	769
11	767	27	783
12	772	28	897
13	775	29	811
14	772	30	778
15	761	31	769
16	711		

(877)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-366
 UNIT NAME Hatch 2
 DATE 11-8-82
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851, X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-51	821001	S	11.3	B	5	NA	RC	CONROD	Load reduction for rod pattern adjustment.
82-52	821002	S	5.3	B	5	NA	HA	TURBIN	Weekly turbine test.
82-53	821008	S	1.0	B	5	NA	RC	CONROD	Load reduction for rod pattern adjustment.
82-54	821008	F	2.0	A	5	NA	HC	HTEXCH	Loss of condenser vacuum.
82-55	821010	S	2.0	B	5	NA	HA	TURBIN	Weekly turbine test.
82-56	821010	S	1.5	A	5	NA	RC	CONROD	Control rod exercises
82-57	821014	F	4.0	A	5	NA	RA	XXX X	ADS inoperable.
82-58	821015	S	4.0	B	5	NA	HA	TURBIN	Weekly turbine testing.
82-59	821022	F	7.5	A	1	NA	SE	XXXXXX	SBGT inop started reactor shutdown.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-366
 UNIT NAME Hatch 2
 DATE 11-8-82
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851, X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-60	821028	F	20.3	B	5	NA	IF	INSTRU	Load reduction to repair "B" RFP instrumentation.
82-61	821029	F	17.0	B	5	NA	CD	VALVEX	Reduced load to repair "B" outboard MSIV.
82-62	821030	F	29.5	B	3	NA	CD	VALVEX	Offload to repair "B" MSIV.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

4
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I - Same Source

NARRATIVE REPORT
UNIT 2

October 1st Load reduction at 2100 hours for rod pattern adjustment.

October 2nd Rod pattern adjustment complete at 0811 hours.

October 2nd Weekly turbine test at 0811 hours.

October 2nd Weekly turbine test complete at 1321 hours.

October 8th Load reduction at 2100 hours for rod pattern adjustment.

October 8th Rod pattern adjustment complete at 2200 hours.

October 8th Loss of condenser vacuum at 2200 hours.

October 10th Weekly turbine test at 0100 hours.

October 10th Weekly turbine test complete at 0300 hours.

October 10th Control rod exercises at 1920 hours.

October 14th ADS inoperable at 1443 hours.

October 14th ADS operable at 1853 hours.

October 15th Weekly turbine testing at 2300 hours.

October 16th Weekly turbine testing complete at 0300 hours.

October 19th SBTG inoperable and started reactor shutdown at 1926 hours.

October 23rd SBTG operable at 0305 hours.

October 28th Load reduction to repair "B" RFP instrumentation at 1103 hours.

October 29th Repair for the above completed at 0715 hours.

October 29th Reduced load at 0715 hours to repair "B" outboard MSIV.

October 30th Offline at 0001 hours to repair "B" MSIV.

October 31st Repair of "B" MSIV completed at 0520 hours.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR October 1982

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-4149	08-11-82	The 5" Ashcroft Bimetal thermometers were installed in (2P41-TI-R384, 2P41-TI-R381A, 2P41-TI-R381B) thermal wells. (Ref: DCR 78-68).
82-969	09-16-82	The (2P52-G001) existing instrument air supply header was removed & a new instrument air supply header was installed at Unit 2 service building annex. (Ref: DCR 80-157).
82-4815	09-14-82	The main steam safety relief valve (2B21-F013H) existing topworks were replaced with spare topworks have a setpoint of 1100 psi. (Ref: DCR 82-202).
82-4688	09-29-82	The (2R43-K774) diesel generator 2A stator high temperature alarm setpoints for all 3 phases were adjusted to 95 ⁰ C. (Ref: DCR 82-167).
82-4687	09-28-82	The (2R43-K775) diesel generator 2C stator high temperature alarm setpoints for all 3 phases were adjusted to 95 ⁰ C. (Ref: DCR 82-167).
82-332	04-24-82	The limit switches (2T48-F118B) in the nitrogen inerting torus makeup INBD. ISO. were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-320	04-30-82	The limit switches (2T46-F001B) in the SBT system filter bed inlet from R.B. were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-313	04-15-82	The limit switch (2T41-F023A) in the RET. FLD. Exh. ISO. damper was replaced with an environmentally qualified limit switch per IEB 79-01B. (Ref: DCR 80-204).
82-296	04-15-82	The limit switches (2D11-F052) in the Fiss. Prod. Mon. containment isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-305	03-13-82	The limit switch (2P33-F007) in the primary containment hydrogen & oxygen containment isolation was replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-311	05-04-82	The limit switches (2T48-F322) in the drywell inerting inboard isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-334	03-10-82	The limit switches (2T48-F321) in the drywell inerting outboard isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-1058	04-14-82	The limit switches (2T48-F337B) in the pressure control valve were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: 80-204).
82-1062	04-24-82	The solenoid valve (2T48-F212) in the drywell-torus dP system was replaced with an environmentally qualified solenoid valve per IEB 79-01B. (Ref: DCR 79-460).

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-4207	08-03-82	The (2B21-N301A) tailpipe pressure switch was installed in place of existing switch per IEB 79-01B.
82-1417	09-23-82	In the hot machine shop floor drain piping (2X45-G001), the associated drain piping & the 150 "S.S.3" buttweld check valve were installed. (Ref: DCR 79-476, R.1).
82-1418	09-23-82	In the hot machine shop floor drain piping, the sump liner per engineering design sketch H-29174 R3B was fabricated & installed. (Ref: DCR 77-476, R1).

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Georgia Power

Edwin I. Hatch Nuclear Plant

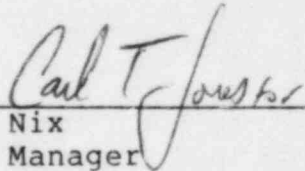
November 15, 1982
PM-82-1193

PLANT E. I. HATCH
NRC Monthly Operating Report

Director
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.



H. C. Nix
Plant Manager

HLS/hh

DUPE

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OPERATING DATA REPORT

SECRET NO. 50-324
DATE 11-10-82
COMPLETED BY BRUNN SMITH
TELEPHONE (912) 367-2701 x.203

 1. Unit Name: 1. Which Nuclear Plant Unit?
 2. Reporting Period: 10-82
 3. Licensed Thermal Power (MWT): 2476
 4. Maximum Available Capacity (MCA): 505.2
 5. Maximum Electrical Rating (MWR): 777.1
 6. Maximum Available Capacity (MCA): 505.2
 7. Maximum Available Capacity (MCA): 777.1
 8. If Changes Occur in Capacity Ratings Since Report, Give Reasons:

9. Power Level to Which Restricted, if Any (Not Used)
 10. Reasons for Restrictions, if Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours in Reporting Period	744	7205	50003
12. Number of Hours Reactor Was Critical	211.2	4575.6	42066.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	212.1	4332.1	40166.3
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	467328	9470911	84218363
17. Gross Electrical Energy Generated (MWH)	141460	3048760	27269490
18. Net Electrical Energy Generated (MWH)	120802	2884097	25093045
19. Unit Service Factor	28.5	59.4	67.1
20. Unit Availability Factor	28.5	59.4	67.1
21. Unit Capacity Factor (Using MCA Net)	23.1	51.9	58.7
22. Unit Capacity Factor (Using MWR Net)	22.6	50.9	58.6
23. Unit Forced Outage Rate	0.0	26.0	20.1
24. Shutdown Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down at End of Report Period, Estimated Date of Startup:
 26. Hours in Last Status (Prior to Commercial Operation):

 INITIAL CRITICALITY
 INITIAL ELECTRICAL
 COMMERCIAL OPERATION

(9277)

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AVERAGE DAILY UNIT POWER LEVEL

PROJECT NO. 50-321
 DATE 11-10-52
 COMPLETED BY OREGON SMITH
 TELEPHONE 251.2. 267-7761 X. 262

MONTH 10-52

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	710	17	-7
2	761	18	-6
3	764	19	-7
4	765	20	-7
5	763	21	-6
6	701	22	-6
7	699	23	-6
8	664	24	-6
9	22	25	-6
10	-17	26	-6
11	-13	27	-5
12	-9	28	-5
13	-9	29	-5
14	-9	30	-5
15	-8	31	-5
16	-7		

(9777)

OPERATING DATA REPORT

COCKET NO. 50-346
 DATE 11-10-82
 COMPLETED BY HARSH SMITH
 TELEPHONE (912) 267-7781 x 203

OPERATING STATUS

- 1 Unit Name: E-1 Hatch Nuclear Plant Unit 2
- 2 Reporting Period: 10-82
- 3 Licensed Thermal Power (MW): 2436
- 4 Available Rating (Gross MW): 917.0
- 5 Net Electrical Rating (Net MW): 734.0
- 6 Maximum Dependable Capacity (MW): 503.9
- 7 Maximum Dependable Capacity (Net MW): 375.9
- 8 Changes occur in capacity ratings every Number 3 through 7. Give Reasons:

- 9 Power level to which restricted, if any (Net MW):
- 10 Reasons for restrictions, if any:

	This Month	Yr -to-Date	Cumulative
11 Hours in Reporting Period	744	7295	27872
12 Number of Hours Reactor Was Critical	725.3	4446.7	19744.9
13 Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14 Hours Generator On-Line	744.0	4111.7	19735.0
15 Net Reserve Shutdown Hours	0.0	0.0	0.0
16 Gross Thermal Energy Generated (MMBtu)	1895836	8545327	40225481
17 Gross Electrical Energy Generated (MMBtu)	527249	2821260	13194710
18 Net Electrical Energy Generated (MMBtu)	514061	2872059	12552584
19 Unit Service Factor	98.1	57.2	67.7
20 Unit Availability Factor	88.5	57.5	63.8
21 Unit Capacity Factor (Using MCR Net)	88.1	46.7	57.9
22 Unit Capacity Factor (Using DER Net)	3.9	22.4	13.2

- 23 Unit forced outage rate
- 24 Unit hours scheduled over next 6 months (Type, Date, and Duration of Each)
- 25 If shut down at end of report period, estimated date of startup
- 26 Units in test status (Prior to Commercial Operation):

INITIAL COMMERCIALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

PROJECT NO. 51-704
DATE 11-10-52
COMPLETED BY RAYON SMITH
TELEPHONE (913) 367-7761 X 403

MONTH 10-52

DAY	AVERAGE DAILY POWER LEVEL (KW)	DAY	AVERAGE DAILY POWER LEVEL (KW)
1	7.47	11	7.53
2	7.53	12	7.84
3	7.53	13	7.82
4	7.71	14	7.81
5	7.73	15	7.80
6	7.75	16	7.80
7	7.77	17	7.72
8	7.79	18	7.84
9	7.81	19	7.83
10	7.82	20	7.83
11	7.87	21	7.83
12	7.89	22	7.87
13	7.89	23	7.87
14	7.90	24	7.87
15	7.90	25	7.87
16	7.91	26	7.87
17	7.91	27	7.87
18	7.91	28	7.87
19	7.91	29	7.87
20	7.91	30	7.87
21	7.91	31	7.87
22	7.91		
23	7.91		
24	7.91		
25	7.91		
26	7.91		
27	7.91		
28	7.91		
29	7.91		
30	7.91		
31	7.91		

(913)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 11-8-62
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851 X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-63	821002	F	2.2	A	5	NA	CB	MOTORX	Loading reduction to repair "A" recirc MG-set brushes.
82-64	821008	F	0.5	A	5	NA	IA	PUMPXX	"B" recirc pump trip.
82-65	821008	S	654.0	C	1	NA	RC	FUELXX	Normal reactor shutdown for refueling outage.

¹ F: Forced
 S: Scheduled

² Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³ Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-366
 UNIT NAME Hatch 2
 DATE 11-8-82
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851, X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-51	821001	S	11.3	B	5	NA	RC	CONROD	Load reduction for rod pattern adjustment.
82-52	821002	S	5.3	B	5	NA	HA	TURBIN	Weekly turbine test.
82-53	821008	S	4.0	B	5	NA	RC	CONROD	Load reduction for rod pattern adjustment.
82-54	821008	F	2.0	A	5	NA	HC	HTEXCH	Loss of condensor vacuum.
82-55	821010	S	2.0	B	5	NA	HA	TURBIN	Weekly turbine test.
82-56	821010	S	1.5	A	5	NA	RC	CONROD	Control rod exercises
82-57	821014	F	4.0	A	5	NA	RA	XXXXXX	ADS inoperable.
82-58	821015	S	4.0	B	5	NA	HA	TURBIN	Weekly turbine testing.
82-59	821022	F	7.5	A	1	NA	SE	XXXXXX	SBGT inop started reactor shutdown.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

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⁵ Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-366
 UNIT NAME Batch 2
 DATE 11-8-52
 COMPLETED BY J. A. Smith
 TELEPHONE 912-367-7851, X-203

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-60	821028	F	20.3	B	5	NA	IF	INSTRU	Load reduction to repair "B" RFP instrumentation.
82-61	821029	F	17.0	B	5	NA	CD	VALVEX	Reduced load to repair "B" outboard MSIV.
82-62	821030	F	29.5	B	3	NA	CD	VALVEX	Offline to repair "B" MSIV.

¹ F: Forced
 S: Scheduled

² Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³ Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

NARRATIVE REPORT
UNIT 1

October 2nd Load reduction at 2000 hours to repair "A"
 recirc MG-set brushes.

October 7th "B" recirc pump tripped at 1205 hours.

October 8th Normal reactor shutdown for refueling at
 2200 hours.

NARRATIVE REPORT
UNIT 2

October 1st Load reduction at 2100 hours for rod pattern adjustment.

October 2nd Rod pattern adjustment complete at 0811 hours.

October 2nd Weekly turbine test at 0811 hours.

October 2nd Weekly turbine test complete at 1321 hours.

October 8th Load reduction at 2100 hours for rod pattern adjustment.

October 8th Rod pattern adjustment complete at 2200 hours.

October 8th Loss of condenser vacuum at 2200 hours.

October 10th Weekly turbine test at 0100 hours.

October 10th Weekly turbine test complete at 0300 hours.

October 10th Control rod exercises at 1920 hours.

October 14th ADS inoperable at 1443 hours.

October 14th ADS operable at 1853 hours.

October 15th Weekly turbine testing at 2300 hours.

October 16th Weekly turbine testing complete at 0300 hours.

October 19th SBGT inoperable and started reactor shutdown at 1926 hours.

October 23rd SBGT operable at 0305 hours.

October 28th Load reduction to repair "B" RFP instrumentation at 1103 hours.

October 29th Repair for the above completed at 0715 hours.

October 29th Reduced load at 0715 hours to repair "B" outboard MSIV.

October 30th Offline at 0001 hours to repair "B" MSIV.

October 31st Repair of "B" MSIV completed at 0520 hours.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR October 1982

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-4149	08-11-82	The 5" Ashcroft Bimetal thermometers were installed in (2P41-TI-R384, 2P41-TI-R381A, 2P41-TI-R381B) thermal wells. (Ref: DCR 78-68).
82-969	09-16-82	The (2P52-G001) existing instrument air supply header was removed & a new instrument air supply header was installed at Unit 2 service building annex. (Ref: DCR 80-157).
82-4815	09-14-82	The main steam safety relief valve (2B21-F013H) existing topworks were replaced with spare topworks have a setpoint of 1100 psi. (Ref: DCR 82-202).
82-4688	09-29-82	The (2R43-K774) diesel generator 2A stator high temperature alarm setpoints for all 3 phases were adjusted to 95°C. (Ref: DCR 82-167).
82-4687	09-28-82	The (2R43-K775) diesel generator 2C stator high temperature alarm setpoints for all 3 phases were adjusted to 95°C. (Ref: DCR 82-167).
82-332	04-24-82	The limit switches (2T48-F118B) in the nitrogen inerting torus makeup INBD. ISO. were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-320	04-30-82	The limit switches (2T46-F001B) in the SBGT system filter bed inlet from R.B. were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-313	04-15-82	The limit switch (2T41-F023A) in the RET. FLD. Exh. ISO. damper was replaced with an environmentally qualified limit switch per IEB 79-01B. (Ref: DCR 80-204).
82-296	04-15-82	The limit switches (2D11-F052) in the Fiss. Prod. Mon. containment isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-305	03-13-82	The limit switch (2P33-F007) in the primary containment hydrogen & oxygen containment isolation was replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-311	05-04-82	The limit switches (2T48-F322) in the drywell inerting inboard isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-334	03-10-82	The limit switches (2T48-F321) in the drywell inerting outboard isolation were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: DCR 80-204).
82-1058	04-14-82	The limit switches (2T48-F337B) in the pressure control valve were replaced with environmentally qualified limit switches per IEB 79-01B. (Ref: 80-204).
82-1062	04-24-82	The solenoid valve (2T48-F212) in the drywell-torus dP system was replaced with an environmentally qualified solenoid valve per IEB 79-01B. (Ref: DCR 79-460).

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
82-4207	08-03-82	The (2B21-N301A) tailpipe pressure switch was installed in place of existing switch per IEB 79-01B.
82-1417	09-23-82	In the hot machine shop floor drain piping (2X45-G001), the associated drain piping & the 150 "S.S.3" buttweld check valve were installed. (Ref: DCR 79-476, R.1).
82-1418	09-23-82	In the hot machine shop floor drain piping, the sump liner per engineering design sketch H-29174 R3B was fabricated & installed. (Ref: DCR 77-476, R1).