OPERATING DATA REPORT

DOCKET NO. 50-289
DATE November 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: October, 1982
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745.	7296.	71569.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	1.0	1.0	840.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	43.6
20. Unit Availability Factor	0.0	0.0	43.6
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	42.5
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	40.7
23. Unit Forced Outage Rate	100.0	100.0	51.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-I
DATE November 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8552

MONTH October, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE November 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

REPORT MONTH October, 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	10/1/82	F	744	D	1				Regulatory Restraint Order

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

OPERATING SUMMARY

The Unit was shutdown the entire report period by order of the NRC. The Reactor Coolant System was partially drained to permit preparation for OTSG repairs. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

During the month of October, restart modification work continued and the following major maintenance items were performed.

The Once Through Steam Generator (OTSG) repair program continued with the following items accomplished:

1. RC-H-1A ("A" OTSG)

   A. Crevis Drying and Dehumidification
   B. Installed Stops/ Bladders in the Hot and Cold Legs
   C. Added Immunol into the Generator
   D. Installed Video Equipment into the Generator
   E. Installed Candles with Detonator Cords into Tubes - First 8 Rows (Total)
   F. Bubbled Immunol Up Tubes to Coat Tubesheet
   G. Explosively Expanded Tubes - Total of 302 Tubes Expanded
   H. Cleaned Tubes/Tubesheet
   I. Performed Profilometry and Eddy Current Tests

2. RC-H-1B ("B" OTSG)

   A. Crevis Drying and Dehumidification
   B. Installed Stops/Bladders in the Hot and Cold Legs
   C. Added Immunol into the Generator
   D. Installed Video Equipment into the Generator
   E. Installed Candles with Detonator Cords into Tubes - First 8 Rows (Total)
   F. Bubbled Immunol Up Tubes to Coat Tubesheet
   G. Explosively Expanded Tubes - Total of 152 Tubes Expanded
   H. Cleaned Tubes and Tubesheets
   I. Performed Profilometry and Eddy Current Tests

This work was performed primarily to test the procedures and hardware and verify sufficient tube expansion prior to continuing with the remaining tubes.

Concentrated Waste Storage Tank (CWST) piping modifications continued with field welding new pipe on the "A" CWST.

The Pressurizer Code Safety Relief Valve modification progressed with the following work items completed:

1. Fit Up Piping ("B" Valve)

2. Welded Piping

3. Radiography Performed - Satisfactorily

4. Reinstalled Missile Shield

5. Commensed Insulation Reinstallation

Local Leak Rate Testing Program commensed with the following valves testing satisfactorily:

1. HR-V-2 A/B, 4 A/B, 22 A/B

2. LR-V-1, 2, 3, 4, 5, and 6

3. RB-V-2A, 7

4. IC-V-18

Repair of IC-V4 was initiated because of high leakage.

The Annual Diesel Generator Inspections on the "A" and "B" diesels were completed with the following items completed:

1. EG-Y-1A ("A" Diesel)

   A. Ran Diesel Engine (Preinspection)
   B. Performed Various Electrical and Mechanical Inspections - Satisfactorily
   C. Performed IC Guage Calibrations
   D. Performed Post Inspection 24 Hour Run - Satisfactorily

2. EG-Y-1B ("B" Diesel)

   A. Ran Diesel Engine (Preinspection)
   B. Performed Various Electrical and Mechanical Inspections - Satisfactorily
   D. Performed IC Guage Calibrations
   E. Performed Post Inspection 24 Hour Run - Satisfactorily

Overhaul of Fire Service Pump (FS-P-2) commensed with the electric motor removed and pump disassembly. Inspection was in progress as of the end of the month.

REFUELING INFORMATION REQUEST

1. Name of Facility:

   Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

   Unknown

3. Scheduled date for restart following refueling:

   Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

   (b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

OPERATING DATA REPORT

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COMMERCIAL OPERATION		

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-I
DATE November 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8552

MONTH October, 1982

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4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October, 1982

DOCKET NO. 50-289
UNIT NAME TMI-1
DATE November 15, 1982
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
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   G. Explosively Expanded Tubes - Total of 302 Tubes Expanded
   H. Cleaned Tubes/Tubesheet
   I. Performed Profilometry and Eddy Current Tests

2. RC-H-1B ("B" OTSG)

   A. Crevis Drying and Dehumidification
   B. Installed Stops/Bladders in the Hot and Cold Legs
   C. Added Immunol into the Generator
   D. Installed Video Equipment into the Generator
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