I A CEPILINICATE PROME	ER D REVISION	NUMBER	E PACKAGE IDENTIFICATION	NUMBER	d PAGE NUMBER	. TOTAL NUMBER P
9186	6		USA/9186/B()F	1	
 PREAMBLE This certificate i of Federal Reguing b. This certificate applicable reguing 	s issued to certify that the packaging and lations, Part 71, "Packaging and Transp does not relieve the consignor from com latory agencies, including the governmi	d contents described cortation of Radioac npliance with any re ent of any country t	d in Item 5 below, meets the tive Material." quirement of the regulation through or into which the p	applicable safet is of the U.S. De backage will be	y standards set for partment of Trans transported.	h in Title 10, Code portation or other
3 THIS CERTIFICATE IS a ISSUED TO /Name	ISSUED ON THE BASIS OF A SAFETY ANA and Address	LYSIS REPORT OF THE	HE PACKAGE DESIGN OR API DENTIFICATION OF REPORT	PLICATION OR APPLICATION		
U.S. Depa Division Washingto	rtment of Energy of Naval Reactors n, DC 20585	Safety Analysis for Shipping S8G Power Units in the S-6213 Container, Rev. 7, dated June 16, 1975, as supplemented.				
		6 DOCKET NU	мыен 71-9186			
4 CONDITIONS This certificate is c	onditional upon fulfilling the requireme	nts of 10 CFR Part	71, as applicable, and the	conditions spec	ified below	
Б.			an ann an t-ann an t-ann a' t-air a' t-air an t-air ann a' t-air ann a' t-air ann a' t-air a' t-air a' t-air a			
(a) Pack	aging					
(1)	Model No .: 5-6213 Pd	ower Unit S	hipping Contair	er		
(2)	Description					
	A power unit shipping complete with control	g container 1 rods and	(PUSC) for shi control rod dri	pment of ve mecha	a power u nisms inst	nit alled.
	The PUSC consists of 9-1/4 feet in outside hemispherical steel of diameter central flar unit is supported in steel plate (PU head flanges of the PUSC of studs. The upper and into the barrel and of the longest control of A lower support adapt container during ship the S6W shipboard poor	a carbon s e diameter end impact nges joinin the PUSC b) which is and fastene d lower ext cover halve rod drive m ter is inst pment of th wer unit.	teel cylindrica by 39-1/2 feet limiters, with g the barrel ar y a centrally 1 clamped betweer d by 94, 2-inch remities of the s without addit echanisms (S8G alled in the ba e S6W prototype	l shell long, in 10-3/4-f d cover ocated t the cen diamete power u ional su Power Un rrel end power u	approximati cluding oot outsid halves. A hick circu tral mating r high stru- nit cantil port exce it Type B of the nit and	ely power lar g ength ever pt for only).
	The PUSC is shipped which is secured to a including frame and for shipments of Type	in the hori a specially contents, w e A and B,	zontal positior built flatbed eighs approxima S8G power units	on a su rail car tely 490	pport fram The PUS ,000 pound	e C, S
	The weight of the PUS 438,900 pounds for sh	SC, includi	ng frame and co the S6W prototy	pe power	s approxim unit and unit	ately

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- 5. (a) Continued
 - (3) Drawings

The Model No. S-6213 PUSC is constructed in accordance with the Drawing Nos. specified in the attachment to this certificate.

- (b) Contents
 - (1) Type and form of materia
 - Unirradiated Naval Reactors Type A or B S8G power unit as described in Chapter 5 of the application and containing uranium enriched in the U-235 isotope.
 - (ii) Unirradiated S6W prototype power unit or unirradiated S6W shipboard power unit as described in Chapter 6 of "S6W Prototype Power Unit in S-6213 Power Unit Shipping Container Safety Analysis Report". WAPD-REO(c)1219, Revision 1, and containing uranium enriched in the U-235 isotope.
 - (2) Maximum quantity of material per package
 - (i) One Type A or Type B S8G power unit.
 - (ii) One S6W prototype power unit or one S6W shipboard power unit.
- (c) Fissile Class

III

Maximum _er of packages per shipment One (1)

- All control rods shall be restrained in the power unit fuel cells by the control rod holddown latches.
- 7. In addition to the requirements of Subpart G of 10 CFR Part 71, a determination shall be made, for each shipment, of the "g" forces that the package or packaging has been subjected to during transport.
 - (a) A nondestructive examination of the entire length of both inner and outer surfaces of the four tie-down support bracket-to-container wall butt welds shall be conducted:
 - if the packaging (with or without contents) has been subjected to "g" forces in excess of 2 g's in any direction through the center of gravity of the package since the last inspection, and

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- 7. Continued
 - (2) following the fourth shipment,* and
 - (3) after every second shipment* following the fourth shipment.

*This requirement shall not be construed to require an inspection if the previous shipment had been inspected in accordance with (7(a)(1)) above.

- (b) The nondestructive examination in accordance with a written procedure may be by either:
 - (1) The liquid penetrant method in accordance with:
 - (i) Article 6, Section V, ASME Code, or
 - MIL-STD-271E, "Nondestructive Testing Requirements for Metals," Section 5, October 31, 1973, or
 - (iii) NAVSHIPS 250-1500-1, "Welding Standard," Section 12.5
 - (2) or the magnetic particle method in accordance with:
 - Article 7, Section V, ASME Code (Yoke Technique; Dry Particle Method; direct or rectified current), or

- MiL-STD-271E, Section 4; specifically 4.3.1 (General) and 5.6.1 (coatings), 4.3.3 (Dry Powder), 4.3.3.3.6 (Continuous), and 4.3.3.3 (Procedure) as excepted by using direct or rectified current, 4.3.3.3.3 (Yoke Technique), 4.3.2.5 (sensitivity and cleaning), and 4.3.1.3 (smoothness), or
- (iii) NAVSHIPS 250-1500-1, Section 12.4, 12.4.1 (General), 12.4.3 (Dry powder), 12.4.3.3.2.1 (Yoke Technique)using direct or rectified current.
- (c) If any indications, as defined in accordance with either:
 - Paragraph UA-93(a), Appendix VIII, Division 1, Section VIII, ASME Code (with 7(b)(2)(i), above), or
 - Paragraphs UA-72 and UA-73, Appendix VI, Division 1, Section VIII, ASME Code (with 7(b)(2)(i), above), or

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- (iii) Class 1 acceptance criteria of NAVSEA 0900-LP-003-8000, "Surface Inspection Acceptance Standards for Metal," with Change 2, July 1, 1974 (with 7(b)(1)(ii) or 7(b)(2)(ii), above), or
- (iv) NAVSHIPS 250-1500-1, Section 10.3.2 (with 7(b)(1)(iii) or 7(b)(2)(iii), above), as noted.

are detected, the packaging shall be repaired and reinspected prior to use and shall be inspected prior to each shipment thereafter. Any defects shall be reported in accordance with 10 CFR §71.95.

8. Expiration date: July 31, 1992

REFERENCES

U.S. Naval Reactors application 'ated July 24, 1975.

Supplements dated: June 3, 1977; and July 24, 1978, Naval Reactors letter G#C89-2838, dated May 22, 1989, and Naval Reactors letter G#C90-03664, dated September 5, 1990.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

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MacDonald, Chief Charles E.

Transportation Branch Division of Safeguards and Transportation, NMSS

JAN 1 6 1991

Date:

ATTACHMENT

The packaging is constructed in accordance with Bingham-Willamette Co. Drawing Nos.:

F-358, Sh. 1 of 1, Rev. A F-372, Sh. 1 of 1, Rev. A F-373, Sh. 1 and 2 of 2, Rev. A F-374, Sh. 1 of 1, Rev. A F-376, Sh. 1 9f 1, Rev. A F-377, Sh. 1 of 1, Rev. A F-404, Sh. 1 of 1, Rev. J F-405, Sh. 1 of 1, Rev. G F-406, Sh. 1 of 1, Rev. J F-408, Sh. 1 of 1, Rev. K F-409, Sh. 1 of 1, Rev. 0 F-424, Sh. 1 of 1, Rev. J F-425, Sh. 1 of 2, Rev. 1 F-425, Sh. 2 of 2, Rev. H F-494, Sh. 1 of 2, Rev. J F-495, Sh. 1 of 2, Rev. M F-495, Sh. 2 of 2, Rev. M F-496, Sh. 1 of 1, Rev. N

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The S8G power unit contents are as shown in Royal Industries, Inc., Drawing No. 130J039, Sh. 1 of 2 Rev. M and General Electric Drawing Nos.: 127D9647, Sh. 1 and 2 of 4, Rev. C 284E809, Rev. O 291E201, Sh. 1 through 3 of 3, Rev. F 291E234, Rev. C 291E246, Sh. 1 and 2 of 2, Rev. C 291E258, Rev. C 291E284, Rev. C 294E810, Sh. 1 of 2, Rev. C 294E811, Sh. 1 of 2, Rev. C 294E812, Rev. C 294E812, Rev. C 294E902, Sh. 1 of 4, Rev. N 294E902, Sh. 2 of 4, Rev. M 294E902, Sh. 3 of 4, Rev. K 294E902, Sh. 4 of 4, Rev. N 294E912, Sh. 1 and 2 of 2, Rev. J 294E930, Sh. 2 of 2, Rev. O

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ATTACHMENT (Continued)

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294E963, Sh. 1 through 3 of 3, Rev. B 294E966, Sh. 2, 4, 5, and 8 of 8, Rev. B 296E204, Rev. B 296E261, Sh. 2 of 2, Rev. C 299E411, Rev. B 299E412, Rev. D 7543E10, Rev. J 7543E23, Sh. 1 of 2, Rev. D 7543E27, Sh. 1 and 2 of 4, Rev. C 7543E29, Sh. 1 and 2 of 3, Rev. C 7543E30, Rev. A 7543E65, Rev. E 7543E65, Rev. E 7543E68, Rev. O

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The S6W prototype power unit contents are shown in Westinghouse Electric Corporation Drawings Nos.:

3D39624, Rev. B 3D39627, Rev. A 1480J01, Rev. C 1480J63, Sheets 1 through 12, Rev. B 1755E07, Sheets 1 through 4, Rev. O 1755E08, Sheets 1 through 6, Rev. O 1927E02, Rev. A 2501F02, Rev. O 2501F12, Rev. O 2501F13, Rev. O 2501F14, Rev. O 8418C26, Rev. O 8418C27, Rev. O 8418C28, Rev. O

The S6W shipboard power unit contents are as shown in Westinghouse Electric Corporation Drawing Nos.:

3D39625, Sheets 1 and 2, Rev. C 3D39626, Rev. B 3D39627, Rev. A 1480J01, Rev. C 1480J88, Sheets 1 through 4, Rev. B 1936F43, Sheets 1 through 8, Rev. J 1582E81, Sheets 1, 2, 4, and 5 of 9, Rev. J 1582E81, Sheets 3, 6, 8, and 9 of 9, Rev. H 1582E81, Sheets 7 of 9, Rev. F

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555

APPROVAL RECORD Model No. S-6213 Power Unit Shipping Container Certificate of Compliance No. 9186 Revision No. 6

By application dated September 5, 1990, Naval Reactors requested that Certificate of Compliance No. 9186 for the Model No. S-6213 Power Unit Shipping Container be amended to include the S6W shipboard power unit as authorized contents.

The S6W shipboard power unit is nearly identical to the S6W prototype power unit, which was previously approved as authorized contents. The total weight of the shipboard package is approximately 1% less than the weight of the prototype package. Because of the similarity of the two packages, many of the analyses performed for the prototype unit package were applicable to the shipboard unit package and were not repeated in the application. The application provided additional analyses to address the structural differences between the two packages, which included a smaller diameter closure head and core barrel for the shipboard power unit.

Based on the structural analyses the only significant damage to the container results from the 30 foot side drop accident. Some localized yielding occurs, but the power unit remains intact and the fuel configuration remains unchanged. During the thirty foot side drop, the inert atmosphere within the container could be lost due to the localized main flange deformation. Loss of the seal of the container would not affect the safety of the shipment. There would be no release of radioactive contents since the fuel remains intact. The loss of the seal does not affect the criticality safety of the package since criticality analyses were performed assuming water inleakage into the package. Criticality analyses were alco performed assuming that the control rods were withdrawn a distance greater than the maximum withdrawal distance which could occur due to hypothetical accident conditions.

Damage to the package due to the puncture accident w_{a} limited to slight deformation of the core barrel, and no contact with the fuel modules was expected. Penetration of the core barrel nozzles by the puncture pin was prevented by protective plates welded to the container at the locations of the nozzles.

The prototype power unit components were evaluated for brittle fracture due to the worst case stresses occurring in any thirty foot drop. The shipboard power unit has the same acceptable design margins against brittle fracture. The fissile loading in the shipboard unit was slightly higher than the prototype unit. The previous criticality analyses for the S6W prototype unit conservatively used the fissile loading of the shipboard unit. The analyses showed that the shipboard unit contents meet the criticality safety requirements for fissile material packages.

The Certificate of Compliance package identification number was changed from USA/9186/AF to USA/9186/B()F. This was a correction in the identification number, since the package had been previously evaluated and approved for Type B quantities of radioactive materials.

These changes do not affect the ability of the package to meet the requirements of 10 CFR Part 71.

Charles Mul

Charles E. MacDonald, Chief Transportation Branch Division of Safeguards and Transportation, NMSS

JAN 1 6 1991

Date

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