Update Report

LICENSEE EVENT REPORT Previous Report Date 12-4-81 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 0 0 0 0 0 - 0 0 3 4 1 LICENSE NUMBER 25 26 0 CON'T 6 6 7 1 1 0 5 8 68 69 EVENT DATE 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On 11-5-81, with Unit 2 shutdown following a Rx. scram, engineering per-0 2 sonnel performing an inspection of electrical conduit discovered that 0 3 RHRSW restraint 2E11-RSW-R26 located on the "B" RHRSW loop in the torus 0 4 room was broken. The AE's stress analysis showed that the pipe would not 0 5 be subjected to stress beyond its design capacity. RHRSW was not declar-0 6 ed inoperable. The health & safety of the public were unaffected due to 0 7 this non-repetitive event. Hatch Unit 2 was not affected by this event. SYSTEM COMPONENT CODE U P O R T B (15) C 13 B (12) CI FI OCCURRENCE REVISION SEQUENTIAL CODE NO. 0 NUMBER COMPONENT ATTACHMENT SUBMITTED NPRD-4 PRIME COMP. HOURS (22 MANUFACTURER A (25) 0 0 0 0 0 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Failure of the support was due to incorrect installation. The damaged welds were repaired & the support correctly installed. The AE performed an analysis & reported the structural integrity of the system to be acceptable in a letter dated 9-2-82. 1 4 METHOD OF (30) DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER 0 0 C Inservice Inspection by AE 80 CONTENT ACTIVITY AMOUNT OF ACTIVITY (35) LOCATION UF RELEASE (36) RELEASED OF RELEASE ] (33) NA 3(34) 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER 10 10 (37) Z (38) NA 80 PERSONNEL INJURIES DESCRIPTION (41) 10 NA 10 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION NA (42) 8211170354 821104 PUBLICITY NRC USE ONLY DESCRIPTION 45 PDR ADDCK 05000366 (44) NA 912-367-7851 H. L. Sumner - Supt. Plt. Eng. Serv. NAME OF PREPARER -

LER #: 50-366/1981-103, Rev. 1 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch

Docket #: 50-366

Narrative Report for LER 50-366/1981-103, Rev. 1 Update Report - Previous Report Date 12-4-81

On 11-5-81, with Hatch Unit 2 shutdown, following a reactor scram due to MSIV closure, Bechtel engineering personnel performing an inspection of electrical conduit discovered that Residual Heat Removal Service Water System pipe support 2Ell-RSW-R26 was damaged. It was determined that failure was due to incorrect installation. The AE recommended that the damaged support be repaired and re-installed as designed. The AE also performed a stress analysis to determine piping integrity. The analysis revealed the piping was not subjected to any stress beyond its design capacity; therefore, the affected loop of RHRSW (the "B" loop) was not declared inoperable. The health and safety of the public were unaffected by this non-repetitive event.

An analysis performed by the AE (B-GP-9589 dated 9-2-82) determined the structural integrity of the system to be unaffected by this event. The previous report incorrectly reported that the system code was SB-"Containment Heat Removal Systems and Controls" rather than CF-"Residual Heat Removal Systems and Controls".