NRC FORM 366 **U. S. NUCLEAR REGULATORY COMMISSION** (7.77) Attachment to AECM--82/547 LICENSEE EVENT REPORT Page 1 of 2 CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) $(\mathbf{1})$ 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 57 CAT 1 20 SGGS 0 0 CON'T L 6 0 5 0 0 0 4 1 6 7 1 0 1 4 8 2 8 1 1 1 1 2 8 2 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 8 REPORT 0 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) When entering Mode 3 (from Mode 4) the requirements of T.S.3.6.5.c were not satisfied 0 2 due to the drywell post-LOCA vacuum breaker position indicators not being operable. 0 3 The inoperability was due to the position indicators having been removed. An LCO was 0 4 entered and the vacuum breakers were verified closed once per 24 hours by local 0 5 indication. The LCO was lifted on October 27, 1982 when the plant re-entered Mode 4. 0 6 This had no effect on the health and safety of the public.and did not constitute a 0 7 threat to plant safety. 80 CAUSE SYSTEM COMP. CAUSE VALVE COMPONENT CODE CODE SUBCODE Z (13) X (12) Z | Z | Z | Z | Z | Z | (16 SIAI ZI 15 REVISION SEQUENTIAL OCCURRENCE REPORT IS 12 1 REPORT NO CODE 0 | 3 TYPE LER/RO Ő 10 17 REPORT L NUMBER ATTACHMENT COMPONENT NPRD-4 PRIME COMP. ACTION FUTURE TAKEN ACTION EFFECT ON PLANT METHOD (22) HOURS FORM SUB SUPPLICE MANUFACTURER 9 9 9 (26) Z 0 0 0 0 Z (23) N (24) X (18) X (19) (20) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 With the plant in Modes 1, 2, or 3, T.S.3.6.5.c requires the drywell post-LOCA vacuum breaker position indicators to be operable. The position indicators were removed to allow proper vacuum breaker operation at the design requirement of 1.0 psid. A Tech. Spec. change has been submitted and results will be reported in a followup report by December 31, 1982. This is an interim report. 4 80 METHOD OF DISCOVERY FACILITY (30) DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER G (28) (31) Surveillance Tesing 0 0 0 B NA 80 ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) RELEASED_OF RELEASE NA Z (33) Z (34) NA 44 80 PERSONNEL EXPOSURES DESCRIPTION (39) TYPE NUMBER (37) Z (38) 0 0 0 NA 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 (40) NA 01 80 8211170314 821112 PDR ADOCK 05000416 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION NA S PDR Z (42) 80 PUBLICITY NRC USE ONLY DESCRIPTION (45) SSUED N (44) NA 68 69 80 10 NAME OF PREPARER Original signed by John M. Albert PHONE:-

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SUPPLEMENTARY INFORMATION TO LER 82-107/03 L-0

Licensee: Mississippi Power & Light Company Facility: Grand Gulf Nuclear Station - Unit 1 Docket No: 50-416

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Prior to the issuance of the facility operating license, Mississippi Power & Light Company (MP&L) requested a revision to the (draft) Technical Specifications on post-LOCA vacuum breakers. This report, MP&L letter AECM-82/186, dated June 10, 1982, advised the NRC Staff of problems associated with these check valves and provided justification for the proposed revision. The NRC Staff was unable to act on the MP&L reguest prior to the formalization of the Technical Specifications with the issuance of the operating license on June 16, 1982. The NRC Project Manager advised MP&L to submit the request through formal channels (10CFR90).

MP&L forwarded a formal request for technical specification revision on this subject in the letter AECM-82/300, dated July 2, 1982. Comments generated by Containment Systems Branch resulted in the MP&L submittal of additional information in AECM-82/441, dated August 5, 1982.

The NRC Staff granted the operating license amendment request in the NRC letter to MP&L, dated October 14, 1982 (Amendment No. 4 to Operating License NPF-13).

MP&L's review of the subject amendment indicated that the Staff's version of the technical specification change as presented in the amendment, was not in accordance with that requested by MP&L in letters dated June 10 and July 2, 1982. The NRC Project Manager was advised of this apparent discrepancy by MP&L in a telephone conversation held November 12, 1982. MP&L contends that the discrepancy is a typographical omission in Amendment No. 4 to the Operating License on the basis that the Staff's safety evaluation report in that amendment approved the deletion of position indication for the subject check valves until the first refueling ovtage.