NRC FOM	M 366	U. S. NUCLEAR REGULATORY COMMISSION
	- •	LICENSEE EVENT REPORT
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02	[On	ctober 12, 1982, at 0914 hours, with the unit in steady state operation at 80%
03	L po	r, valve CVC-203 (Letdown Relief Valve) lifted and did not reseat completely.
04	LTh	resulting leakage was determined to be approximately 1.6 GPM which is in excess
0 5	of	the allowable leakage defined by Technical Specification 3.1.5.1 and is reported
06	Lpu	uant to 6.9.2.b.2. The leakage was confined to the Pressurizer Relief Tank
07	Lan	Containment; thus, there was no threat to the public health and safety.
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10	LNo	lefinite cause for the lifting of relief valve CVC-203 has been determined.
11	LTH	failure to completely reseat is believed to be the result of valve seat
12	Lda	age caused by valve chatter during the lifting of the valve. CVC-203 was
13	Lre	aired and returned to service. As stated in LER-82-12, investigation into
14	Lfr	reliability of CVC-203 is continuing, and any additional actions resulting a these reviews will be reported as a supplement to that report.
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SUPPLEMENTAL INFORMATION

FOR

LICENSEE EVENT REPORT 82-014

I. Cause Description and Analysis

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On October 12, 1982, at 0914 hours, with the unit in steady state operation at 80% power, the Low Pressure Letdown Relief Line High Temperature Alarm was received on the control board. Investigation revealed that valve CVC-203 (Letdown Relief Valve) had lifted and had not completely reseated. The resulting primary system leakage was determined to be approximately 1.6 GPM.

No definite cause for the lifting of Relief Valve CVC-203 has been determined. However, a combination of system design and valve design problems is believed to be the root cause of this event. The failure of the valve to completely reseat was a result of seat damage caused by valve chatter during the lifting of the valve. This event resulted in an identified primary system leak in excess of 1 GPM as defined by Technical Specification 3.1.5.1 and is reported pursuant to 6.9.2.b.2. The leakage was routed to the Pressurizer Relief Tank via the Letdown Relief Line with only slight leakage to the Containment Vessel floor from the valve weep hole. This event did not result in a threat to the public health and safety because the leakage was contained and capability existed, throughout the event, to isolate the valve and bring the plant to hot shutdown.

II. Corrective Action

Upon identifying that CVC-203 was not completely seated, the letdown line pressure was reduced, and the valve was manually reseated reducing total reactor system leakage to approximately .6 GPM. The maximum leak rate during this event war 1.6 GPM.

With the primary system leakage within acceptable limits, plant operation continued until 2251 hours on October 22, 1982, when the unit was placed in hot shutdown to manually isolate CVC-203 for repairs. CVC-203 was repaired by replacing the entire valve internals with new components. Also, in an effort to identify some potential system problems which could have contributed to the lifting of CVC-203, PCV-145 (Letdown Pressure Control Valve) was inspected, and some minor adjustments were made. These adjustments were not of sufficient magnitude to have caused this event. The unit returned to power operation at 0459 hours on October 24, 1982, with CVC-203 and PCV-145 operating satisfactorily.

III. Corrective Action to Prevent Recurrence

As stated in a previous LER (LER-82-12) concerning CVC-203, investigation into the reliability of CVC-203 and efforts toward resolution are in progress. An engineering review of the CVCS Letdown System design and operation has also been initiated. Any additional actions resulting from these reviews will be reported as a supplement to LER-82-12.