NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

(7.77)	LICENSEE EVENT REPORT	EXHIBIT A
	CONTROL BLOCK:	TIONI
0 1	F L C R P 3 2 0 0 - 0 0 0 0 0 0 0	CAT 58
0 1 7 8	SOURCE 60 61 DOCKET NUMBER 64 69 EVENT DATE 74 75 REPORT DATE	32
0 2	LAt 1245 on 6/24/81, FPC was notified by Henry Pratt Co. that conta	
0 3	purge valves AHV-1A.1B,1C, and 1D were in a condition which could	
04	to fail during a LOCA. This is contrary to T.S. 3.6.1.1 and T.S.3	.6.3.1.
0 5	Operability was restored at 1340 on 7/1/81. At 0930 on 11/1/82, Gi	
0 6	Associates, Inc. informed FPC that containment purge valves, when	in the
0 7	full open position could allow dose rates to exceed the 10CFR 100	
0 8 7 8	This was the third event under T.S.3.6.1.1. and the thirty-sixth event under T.S.3.6.3.5.3.5.3.5.3.5.3.5.3.5.3.5.3.5.3.5	ent under
0 9	S D TO B TO SUBCODE SU	
	TO REPORT NUMBER 21 22 23 24 25 25	NO.
	ACTION FUTURE OFFECT SHUTDOWN METHOD HOURS 22 ATTACHMENT NPRO4 PRIME COMP. SUPPLIER SUBMITTED FORM SUB. PRIME COMP. SUPPLIER SUPPLIER SUBMITTED FORM SUB. PRIME COMP. SUPPLIER SUBMITTED FORM SUB. SUPPLIER SUPPLI	COMPONENT ANUFACTURER
	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The cause of this event is not known. On 7/1/81, the Pratt valves	47
10	as recommended by H. Pratt Co to 50° for Bettis actuators and 35°	
11		
1 2	Limitorque operators. The modifications also resolved the 10CFR 1	oo dose
13	L rate excess identified by Gilbert Associates, Inc. on 11/1/82.	
7 8		
15	F 28 10 12 13 Wender Notification 32) "
	CONTENT CONTEN	
17	NUMBER O 37 Z 38 DESCRIPTION A	80
18	PERSONNEL INJURIES 13 NUMBER DESCRIPTION (1) N/A	80
10	LOSS OF OR DAMAGE TO FACILITY 43 N/A	80
20	PUBLICITY SSUED DESCRIPTION (45) N/A NRC US N/A	E ONLY
2 8	D G Higher foll (90%) 57 95-380	2 80
	P. G. Hughes	

SUPPLEMENTARY INFORMATION

REPORT NO: 50-302/81-031/01T-1

FACILITY: Crystal River Unit #3

REPORT DATE: July 8, 1981 (Rev. 0)/November 15, 1982 (Rev. 1)

OCCURRENCE DATE: June 24, 1981 (Rev. 0)/November 1, 1982 (Rev. 1)

IDENTIFICATION OF OCCURRENCE:

Containment Isolation Valves AHV-1A, IB, IC and ID were operating in a condition less conservative than the vendors' reevaluations for safety-related application. This condition was contrary to Technical Specifications 3.6.1.1 and 3.6.3.1 and Final Safety Analysis Report, Section 14.2.2.7.2.

CONDITIONS PRIOR TO OCCURRENCE:

MODE 1 (100% Full Power)

DESCRIPTION OF OCCURRENCE:

At 1245 on June 24, 1981, Henry Pratt Company informed Florida Power Corporation that the previously established operating limit of 60° on containment isolation valves AHV-1A, 1B, 1C and iD was not adequate for LOCA conditions. The engineering evaluation to determine adequacy of the valves was initiated as a result of a 1979 commitment to the NRC. This evaluation determined that the valve structure and actuator were inadequate to withstand LOCA-reduced loads based on the torques calculated in the evaluation. The Pratt evaluation recommended partially closing the valves to 35° and 50° to relieve the over-torque expected to occur during a LOCA.

On June 24, 1981, the Reactor Building purge that was in progress was terminated at 1335, and the valves were deactivated pending implementation of an Engineering resolution to modify the valves as recommended.

At 0930 on November 1, 1982, Gilbert Associates, Inc., informed Florida Power Corporation that the original 90° operating limit on containment isolation valves AHV-1A, 1B, 1C and 1D would have allowed the post-LOCA dose rate to exceed 10 CFR 100 limits. Thus, prior to the initial modification to resolve the torquing concerns above, Crystal River Unit 3 was operating in a condition less conservative than assumed in the Safety Analysis, Section 14.2.2.7.2.

DESIGNATION OF APPARENT CAUSE:

These events were caused by apparent oversights in the design process.

ANALYSIS OF OCCURRENCE:

There was no effect on public health.

CORRECTIVE ACTION:

The valves were modified to comply with the recommended operating limits. Operability was restored at 1340 on July 1, 1981.

FAILURE DATA:

This is the thirty-sixth event reported under Technical Specification 3.6.3.1, and the third under Technical Specification 3.6.1.1.