

x

SACRAMENTO MUNICIPAL UTILITY DISTRICT C P. O. Box 15830, Sacramento CA 95852-1830, (916) 452 3211 AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

AGM/NUC 91-005

January 17, 1991

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Ducket No. 50-312 Rancho Seco Nuclear Generating Station License No. DPR-54 LICENSEE EVENT REPORT 90-03: EXCEEDING THE DESIGN BASIS OF THE PLANT DUE TO EXTREME LOW OUTSIDE TEMPERATURES.

Attention: Seymour Weiss

In accordance with the requirements of 10 CFR Part 50.73(a)(2)(ii)(6) the Sacramento Municipal Utility District hereby submits Licensee Event Report Number 90-03.

Members of your staff with questions requiring additional information or clarification may contact Robert Jones at (209) 333-2935, extension 4676.

Sincerely,

Dan R. Keuter Assistant General Manager Nuclear

Attachment

cc w/atch: J. B. Martin C. Myers, NRC, Rancho Seco

9101220168 910117 PDR ADOCK 05000312 PDR ADOCK PDR S

1322

NRC Form 366 19 831	alama ayo ak ada qona a	LICE	ENSEE EVEN	T REF	PORT	(LER)	U.S. NU	CLEAR REGULI APPROVED OM EXPIRES 8/31/1	NO 3150- S	10104
Rancho Seco Nucl	ear Gener	ating St	ation			D	OCKET NUMBER	(2) (1) (2) (1) (1)	2 1 0	101 10
Exceeding the De	sian Rasi	e of the	Diant Due				191010	0 3 11 1	2 1 0	F[0] -
saccounty the be	sign basi	s or the	Plant Due	tot	xtren	ne Low Out	side Tem;	perature	S	
MONTE DAY YEAR YEAR	SEQUENTIAL	B) REVENON	REPORT DATE	(P) VEAE		PACILITS NAM	ACILITIES INVOL	VED (8) DOCKET NUME	ER(s)	
	NUMBER	NUMBER .						0 15 10 1	0101	1.1
12 27 90 90	- 0 0 3	- 0 0	0 1 1 7	91				0 15 10 1	0 1 0 1	
OPERATING N THIS REP	ORT IS SUBMITTE	D PURBUANT TO	D THE REQUIREMEN	TE OF 10	CFR & IS	Check one or more of	the following) (11	1		
20.4 POWER 20.4	402 (b.) 406 (a.) (5.) (i)		20.405(c)			50 73(a)(2)(iv)	영문 위 경	73.71(b)		
LEVEL Q 0 0 20 1	405(a)(1)(ii)	k =======	60.36(c)(2)		-	50.73(a)(2)(vii)		OTHER	Specify in A	Datract
20.1	406 (a) (1) (in)		50.73(a)(2)(i)			50.73(a)(2)(am)(A)		below an 366A)	t in Taxt, NI	RÇ Form
20.4	406(a)(1)(iv)	X	80.73(a)(2)(0)			80.73(e)(2)(sib)(8)				
20 /	605(a)(1)(v)		60.73(a)(2)(iii)	0.0		80.72(a)(2)(x)				
nan den staten in den staten sta			GENBEE CONTACT I	OR THIS	LER (12)			TELEPHONE NO	IMBER	
Robert E. Jones,	Licensing	g Engine	er				9 1 6	452	- 3	2 1 1
	COMPLETE	ONE LINE FOR	EACH COMPONENT	FAILURE	DESCRIBE	D IN THIS REPORT	(13)			
CAUSE SYSTEM COMPONENT	NANUFAC TURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MENUFAD TURER	REPORTABL TO NPRDS	F	********
	111									
	111				1		111			
an a	SUPPLEME	NTAL REPORT	EXPECTED (14)			Construction being and the sead	EXPECTE	MON	TH DAY	YEAR
VES III AN ADMINAR EVERATED	NIDAIREICIN CATE						SUBMISSIO DATE IN	DN .		
The minimum de December 22, 1 temperature at low as 15°F. Suppression Zc and capable of Approximately ability of the The District f leakage or phy corrected, as leak from the offsite releas As required by compensatory m Engineering ev additional phy	esign temp parts the site System was one 35A, as fulfilli 75 leaks e affected formed tea vsical dam appropria Radwaste e of radi the Fire easures f aluate th sical wor	perature n approxi e dropped lkdowns ll requi ng their were ide systems ms to co age to p te, usin System r oactive Protect or Fire e effect k is req tures di	at Rancho mate.y 030 below the determined red plant function entified; H to perfor onduct systemes an into a liquid. ion Plan, Suppression s of the r uired for	Seco DO hou e 19° d that systi in the nowever rm the cem water the Second storm the Second the Second the Second the Second the Seco	is 1 F des t, wi ems w he lo er, ti alkdou y par iest p d ta shift ie 35/ ed ten ted p	9°F (dry b o 0900 hou ign basis th the exc ere suitab ng-term de he damage ntended fu wns and do rts. Dama process. in resulti Superviso A. nperatures plant syst	oulb). Ours, the temperat eption o le for c fueled c did not inction. cument a ge is be In addit ng in a n r implement to deten ems.	n outdoor ure to a f Fire ontinued ondition affect t 11 signs ing ion, a 2 minor ented rmine if	as d use t. the of gpm	

*

State and

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104

YDIDES 8:31/00	

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)	
		YEAR SEQUENTIAL REVISION NUMBER NUMBER		
Rancho Seco Nuclear Generating Station	0 5 0 0 0 3 1 2	910 -01013-010	0 12 OF 0 4	

Description of the Event

NRC Form 366A

Updated Safety Analysis Report (USAR), Appendix 2B "Meteorology" lists the historical minimum temperature in the area of Rancho Seco as 19°F. In addition, Nuclear Engineering Procedure NEP 5101.1 "Site Information" states that the minimum design temperature at Rancho Seco is 19°F (dry bulb).

On December 22, 1990, from approximately 0300 hours to 0900 hours, the outdoor temperature at the site dropped below 19°F to as low as 15°F. System walkdowns determined that, with the exception of Fire Suppression Zone 35A, all required plant systems were suitable for continued use and capable of fulfilling their function in the long-term defueled condition. Approximately 75 leaks were identified; however, the damage did not affect the ability of the affected systems to perform their intended function.

One leak resulted in a minor offsite release of radioactive water. A 2 gpm body to bonnet leak on Radwaste System valve lasted approximately 4 hours before being isolated. The leakage flowed through a drain in the north wall of the Tank Farm and into a storm drain located south-east of the east cooling tower. The leakage then travelled offsite to a field south of the plant.

Gamma spectroscopy and tritium analysis results showed radioactivity levels of 3.84E-2 uCi/ml H-3 and 3.59E-8 uCi/ml Cs-137. The total estimated release volume was 1.82E6 ml and the total whole-body dose (child) was 7.64E-4 mrem. This release, along with its associated contribution to offsite dose, will be discussed in the next semi-annual effluent report.

Additional leakage occurred on January 14, 1991, when a Service Water System (SWS) line in the Tank Farm broke after Operations returned the system to service after having previously isolated the system due to freeze damage. Although SWS is not a contaminated system, some leakage became contaminated (3.11E-8 Cs-137 and 9.05E-9 Co-60) when it collected in an area immediately adjacent to the Spent Fuel Pool cooler bermed area. This leakage subsequently soaked into the ground. Some water may have leaked into a sealed storm drain; however, Operations had diverted the storm drain to the retention basin, thus there was no abnormal offsite release of radioactive liquid.

Plant Operation Conditions

Rancho Seco has been shutdown since June 7, 1989, and has been defueled since the last of the fuel was removed on December 8, 1989.

Cause of the Event

The District established the minimum design temperature for Rancho Seco based on a review of historic meteorological data. The record setting low temperatures were an act of nature.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

PAGE (3)

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88

LER NUMBER (6)

EAR

FACILITY NAME (1)	DOCKET NUMBER (2	}

NUMBER NUMBER 3 OF 0 5 0 0 0 3 1 2 Rancho Seco Nuclear Generating Station 90 00 0 0 0 04 3 TEXT (If more spece is required, use additional NRC Form 3664's) (17)

Energy Industry Identification System (EIIS) Component and System Identifier

The EIIS system identifier for Fire Protection is KP.

The EIIS system identifier for Spent Fuel Cooling is DA.

The EIIS system identifier for the Radwaste System is WD

Method of Discovery

NRC Farm 386A

Local weather reports and data from the National Weather Bureau indicated that temperatures in the Sacramento and Stockton area dropped below 19°F. Subsequent review of site meteorological data during the period from December 20, 1990, through December 27, 1990, indicated that the temperature dropped below 19°F for approximately 6 hours on December 22, 1990.

Safety Consequences

The District conducted an evaluation to determine if the abnormally low temperatures resulted in an unreviewed safety question. Given Rancho Seco's permanently defueled condition, the only credible design basis accidents are those associated with the loss of spent fuel pool water or with fuel handling.

Fuel handling equipment is located indoors and was not exposed to the extreme low temperatures. The spent fuel cooling system was operating, moving warm water through system piping, and was protected from the low temperatures. Other systems protecting spent fuel, including component cooling water and plant cooling water, were operating during the extreme cold weather, thereby surficiently heating and protecting these systems. In addition, redundant systems remained operable/available, as required.

The extreme low temperatures did not result in an unreviewed safety question. There were no health or safety consequences as result of this event.

Corrective Actions

In accordance with 10 CFR 50.72(b)(1)(ii)(B), the Shift Supervisor initiated a one-hour telephone notification to the NRC.

The District formed teams to conduct walkdowns of those portions of required systems which were located outside and which contained water. The teams consisted of a senior systems engineer, a licensed operator, and an I&C or electrical technician, as appropriate. The teams documented all signs of leakage or physical damage to pressure boundary parts. Damage is being corrected as appropriate, using the work request process.

As required by the Fire Protection Plan, the Shift Supervisor implemented compensatory measures for Fire Suppression Zone 35A.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OM8 NO. 3150-0104 EXPIRES 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAQE (3)	
		YEAR SEGUENTIAL REVISION NUMBER NUMBER		
	6 15 10 10 10 10 10 10 10 10		- 1 - 05 - 1-	
Rancho Seco Nuclear Generating Station	0 0 0 0 3 1 2	90-003-000	0 4 10 10 14	

TEXT (If more space is required, use additional NRC Form 3064/s) (T)

With the exception of SWS, all systems that were depressurized to repair leaks have been repressurized with no additional leaks identified. Operations will restore 5.5 to service in sections, with individuals observing the area being restored to ensure rapid detection and isolation of leaks.

Engineering will evaluate the effects of the reduced temperature to determine if additional physical work is required for affected plant systems.

Chemistry prepared an Abnormal Liquid Release Report and will discuss the abnormal liquid release in the next semi-annual effluent report.

Previous Similar Events

NRC Korm 366A

A review of previous LERs revealed that there were no previous occurrences where the ambient temperature fell below the design basis temperature.