

OPERATING DATA REPORT

DOCKET NO. 50-368  
 DATE 08/14/81  
 COMPLETED BY L. S. Bramlett  
 TELEPHONE 501-968-2519

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: July 1, - 31, 1981
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
None

9. Power Level To Which Restricted, If Any (Net MWe): None  
 10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>5087.0</u>	<u>11831.0</u>
12. Number Of Hours Reactor Was Critical	<u>676.2</u>	<u>2669.1</u>	<u>7699.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>54.9</u>	<u>859.5</u>
14. Hours Generator On-Line	<u>569.3</u>	<u>2521.5</u>	<u>7437.5</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>75.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>951509.0</u>	<u>6046394.0</u>	<u>17890556.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>295888.0</u>	<u>1969477.0</u>	<u>5802378.0</u>
18. Net Electrical Energy Generated (MWH)	<u>277335.0</u>	<u>1877240.0</u>	<u>5524437.0</u>
19. Unit Service Factor	<u>76.5</u>	<u>49.6</u>	<u>62.9</u>
20. Unit Availability Factor	<u>76.5</u>	<u>49.6</u>	<u>63.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>43.4</u>	<u>43.0</u>	<u>54.4</u>
22. Unit Capacity Factor (Using DER Net)	<u>40.9</u>	<u>40.5</u>	<u>51.2</u>
23. Unit Forced Outage Rate	<u>9.4</u>	<u>6.4</u>	<u>21.2</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_  
 26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-368

UNIT 2

DATE 08/14/81

COMPLETED BY L. S. Bramlett

TELEPHONE 501-968-2519

MONTH July

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	409
2	0	18	377
3	0	19	306
4	0	20	402
5	6	21	382
6	31	22	632
7	128	23	633
8	371	24	638
9	384	25	628
10	398	26	616
11	405	27	627
12	406	28	635
13	310	29	651
14	0	30	681
15	271	31	824
16	406		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

REFUELING INFORMATION

DATE: July, 1981

1. Name of facility. Arkansas Nuclear One - Unit 2

2. Scheduled date for next refueling shutdown. 9/1/82

3. Scheduled date for restart following refueling. 11/1/82

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
If answer is yes, what, in general, will these be?  
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes. Description of effects of new core loading.  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

5. Scheduled date(s) for submitting proposed licensing action and supporting information. 6/1/82

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Possible utilization of Core Protection Calculator (CPC) semi-addressable constants.  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 60

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
present 485 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1989

NRC MONTHLY OPERATING REPORT  
OPERATING SUMMARY - JULY 1981  
UNIT II

The unit began the month in mode 2 while performing low power physics testing. The unit was limited to one feedwater pump operation because "B" pump turbine was destroyed during functional testing in June. On July 1, 1981, the reactor was manually tripped for physics testing and was returned to criticality later that day. On July 3, 1981 the reactor tripped on low steam generator level while in mode 2 operation and was returned to operation later that day. On July 4, 1981, the reactor was manually shutdown in accordance with technical specification 3.6.1.4 (Containment pressure, temperature and humidity limits). The reactor was brought critical on July 5, 1981 after maintenance was performed on the 1-B main-chiller and the containment building was brought within the limits of T.S. 3.6.1.4. Later that day the unit was tied on line and tripped off on reverse power. Again the unit was tied on line, however later that day it was taken off line for maintenance on a main feedwater block valve. The unit was tied on line on July 6, 1981 and tripped on July 7, 1981 due to an inadvertant reverse power turbine/generator trip, with reactor power manually reduced to approximately 9%. Later that day while increasing reactor power to approximately 13% the reactor tripped on high steam generator level (Generator "A") due to sensitive feedwater flow control at less than 15% power. The reactor was brought critical and the unit placed on line later that day. During power escalation to 50% full power the unit experienced several turbine runbacks due to the stator coolant flow switch being out of calibration. On July 8, 1981 the unit reached approximately 50% full power for further physics testing. On July 13, 1981 the unit was manually tripped when "A" MFW pump speed reduced drastically due to a speed sensing circuit board failure.

The unit was returned to 50% power on July 15, 1981 for continued physics testing. On July 18, 1981 the unit tripped on high LPD and low DNBR due to a combined penalty factor induced when PLCEA #26 dropped during physics testing. The unit was returned to 50% power on July 19, 1981 for completion of physics testing. On July 21, 1981, power was reduced to 35% for a capacity test on an atmospheric steam dump valve and was returned to 50% power with escalation up to approximately 75% power. Power was limited to approximately 75% power due to a condenser hotwell temperature limit of 135°F. This limit was reached due to one circulating water pump being in maintenance. On July 23, 1981 a functional test of an atmospheric steam dump was performed by holding reactor power constant and decreasing turbine capacity approximately 13%. This was repeated on July 24, 1981.

On July 26, 1981 reactor power was reduced to 60% power for a capacity test on an atmospheric steam dump valve and was later returned to approximately 75% power. On July 30, 1981 the circulating water pump was returned to service and reactor power was increased to 90%. On July 31, 1981 the reactor was brought to 99% power to complete the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-368  
 UNIT NAME ANO - Unit II  
 DATE 8-6-81  
 COMPLETED BY L. S. Bramlett  
 TELEPHONE (501) 968-2519

REPORT MONTH July

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-06	810328	S	87.7	C	4	None	ZZ	ZZZZZZ	Refueling
	810701	S	1.5	B	1	None	ZZ	ZZZZZZ	Low Power Physics Testing
	810703	F	2.2	H	3	None	ZZ	ZZZZZZ	Low Steam Generator Level
	810704	F	24.5	F	1	50-368/024	ZZ	ZZZZZZ	T.S. 3.6.1.4
81-07	810705	F	.2	H	3	None	ZZ	ZZZZZZ	Reverse Power Trip
81-08	810705	F	13.8	B	1	None	CH	VALVEX	Maintenance on FW Block Valve
81-09	810707	F	2.7	H	3	None	ZZ	ZZZZZZ	Reverse Power Trip
	810707	F	1.9	H	3	None	ZZ	ZZZZZZ	High Steam Generator Level
81-10	810713	F	35.1	A	1	None	CH	INSTRU	Failure of circuit board in "A" MFW pump speed sensing circuit
81-11	810718	F	5.1	B	3	None	ZZ	ZZZZZZ	PLCEA #26 Dropped

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuation  
 5-Load Reduction  
 9-Other

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source