

OPERATING DATA REPORT

DOCK#T NO. 50-368
 DATE 12/14/81
 COMPLETED BY Bramlett
 TELEPHONE 501-954-3145

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: November 1-30, 1981
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 942.57
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): None

10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.0</u>	<u>8016.0</u>	<u>14760.0</u>
12. Number Of Hours Reactor Was Critical	<u>700.2</u>	<u>5136.2</u>	<u>10167.0</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>209.1</u>	<u>1013.7</u>
14. Hours Generator On-Line	<u>690.2</u>	<u>4917.4</u>	<u>9833.4</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>75.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1664622.</u>	<u>12090619.</u>	<u>23934781.</u>
17. Gross Electrical Energy Generated (MWH)	<u>538426.</u>	<u>3939801.</u>	<u>7772702.</u>
18. Net Electrical Energy Generated (MWH)	<u>511284.</u>	<u>3752214.</u>	<u>7399411.</u>
19. Unit Service Factor	<u>95.9</u>	<u>61.3</u>	<u>66.6</u>
20. Unit Availability Factor	<u>95.9</u>	<u>61.3</u>	<u>67.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>82.8</u>	<u>54.6</u>	<u>58.4</u>
22. Unit Capacity Factor (Using DER Net)	<u>77.9</u>	<u>51.3</u>	<u>55.0</u>
23. Unit Forced Outage Rate	<u>4.1</u>	<u>12.6</u>	<u>20.5</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-368
 UNIT 2
 DATE 12/14/81
 COMPLETED BY Bramlett
 TELEPHONE 501-964-3145

MONTH November

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	593
2	861
3	883
4	482
5	97
6	291
7	376
8	371
9	411
10	713
11	953
12	874
13	873
14	872
15	872
16	872

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	876
18	875
19	876
20	877
21	874
22	873
23	468
24	737
25	872
26	872
27	412
28	645
29	871
30	869
31	N/A

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

NRC MONTHLY OPERATING REPORT
OPERATING SUMMARY - NOVEMBER, 1981
UNIT 2

The Unit began the month at 60% power for checkout and operation of "B" Main Feedwater Pump. On 11-2-81 the Unit reached 100% power operation. The Unit operated at 100% until the reactor tripped on 11-4-81 due to a Channel 3 and 4 low DNBR trip. The low DNBR was traced to an anomalous T_H indication. The Unit was returned to Mode 1 on 11-5-81, however, the reactor tripped on ASI limits. The Unit was again returned to Mode 1 on 11-5-81 and remained at approximately 50% power while searching for condenser tube leaks until 11-9-81. 100% power operation was obtained on 11-11-81 and the Unit operated there for 12 consecutive days. On 11-23-81 the Unit was tripped due to an I&C technician inadvertently causing the supply breaker for "A" Steam Generator level to trip. The Unit reached 100% power operation on 11-24-81. On 11-27-81 the Unit tripped on a spurious control signal causing a feedwater flow upset. Later that day Mode 1 was reached and the Unit was manually tripped when both Main Feedwater Pumps tripped. The Unit was returned to 100% power operation on 11-28-81 and operated there the remainder of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November

DOCKET NO. 50-368
 UNIT NAME ANO-II
 DATE 12-3-81
 COMPLETED BY L. S. Bramlett
 TELEPHONE (501) 964-3145

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-28	811104	F	12.7	H	3	None	ZZ	ZZZZZZ	Unit trip on low DNBR due to Anomalous T _H inputs to the CPC's.
81-29	811105	F	2.1	H	3	None	ZZ	ZZZZZZ	Unit trip on high LPD due to ASI.
81-30	811123	F	5.1	G	3	None	ZZ	ZZZZZZ	Unit trip on low S/G level when the level instrumentation's power supply tripped. (Human Error)
81-31	811127	F	5.3	H	3	None	ZZ	ZZZZZZ	FW Flow upset; spurious
81-32	811127	F	4.6	H	1	None	ZZ	ZZZZZZ	FW pumps tripped; Rx manually tripped. Spurious

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation
 5-Load Reduction
 9-Other

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

REFUELING INFORMATION

DATE: November 1981

1. Name of facility. Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. 9/1/82
3. Scheduled date for restart following refueling. 11/1/82
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
If answer is yes, what, in general, will these be?
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes. Description of effects of new core loading.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. 6/1/82
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
Possible utilization of Core Protection Calculator (CPC)
semi-addressable constants.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 60
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 485 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: 1989