#### OPERATING DATA REPORT

DOCKET NO.
DATE
COMPLETED BY
TELEPHONE

DOCKET NO.
DATE
D/12/82
Bramlett
501-964-3145

OPERATING STATUS		-	
L Unit Name: Arkansas Nuclear One -	Unit 1	Notes	7 7 7 7 7
December 1 - 31 19	81		31500 1500
Reporting rende			
3. Licensed Thermal Power (MWt): 2568 4. Nameplate Rating (Gross MWe): 902.74			
4. Nameplate Kating (Gross Mine)			
5. Design Electrical Rating (Net MWe)	883		
o, maximum be pendable capacity to the	836		
8. If Changes Occur in Capacity Ratings (Items Num		ince Last Report Give Re	easons:
None None	The state of the s		
9. Power Level To Which Restricted, If Any (Net M	Wel: None	*	
10. Reasons For Restrictions, If Any: N/A			
	This Month	Yrto-Date	Cumulative
			61675 0
11. Hours In Reporting Period	744.0	8760.0	61675.0
12. Number Of Hours Reactor Was Critical	744.0	6457.2	42247.5
13 vReactor Reserve Shutdown Hours	0.0	149.0	5044.0
14. Hours Generator On-Line	744.0	6338.8	41383.8
15. Unit Reserve Shutdown Hours	0.0	-	
16. Gross Thermal Energy Generated (MWH)	1713511.	15358240.	99530206.
17. Gross Electrical Energy Generated (MWH)	578550.	5138195.	32836756.
18. Net Electrical Energy Generated (MWH)	550978.	4900762.	31316401.
19. Unit Service Factor	100.	72.4	68.4
20. Unit Availability Factor V	100.	72.4	
21. Unit Capacity Factor (Using MDC Net)	88.6	66.9	60.7
22. Unit Capacity Factor (Using DER Net)	87.1	65.8	59.7 16.1
23. Unit Forced Outage Rate	0.0	0.9	10.1
<ol> <li>Shutdowns Scheduled Over Next 6 Months (Type None</li> </ol>	. Date, and Duration	n of Each):	
25. If Shut Down At End Of Report Period, Estimate	ed Date of Startun		
26. Units In Test Status (Prior to Commercial Operat		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

## AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-313
UNIT	1
DATE	1/12/82
COMPLETED BY	Bramlett
TELEPHONE	501-964-3145

MON	TH December		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	713	17	748
2	713	18	744
3	712	19	745
4	713	20	745
5	715	21	744
6	736	22	743
7	764	23	742
8	762	24	738
9	758	25	736
10	764	26	732
11	760	27	734
12	757	28	734
13	755	29	735
14	751	30	734
15	751	31	735
16	748		

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

50-313 DOCKET NO. ANO-Unit UNIT NAME DATE January 4, 1982 COMPLETED BY TELEPHONE (501) 964-3155

# REPORT MONTH December

No.	; Date	Type1	Duration (Hours)	Reason 2	Method of Shutting Down Reactor3	Licensee Event Report #	System Code <sup>4</sup>	Component Code 5	Cause & Corrective Action to Prevent Recurrence
NONE									

F: Forced S: Scheduled

(9/77)

Reason

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

D-Regulatory Restriction E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain) H-Other (Explain)

Method:

1-Manual

2-Manual Scram.

3-Automatic Scram. 4-Continuation

5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit 1 - Same Source

#### UNIT SHUTDO' NS AND POWER REDUCTIONS

#### INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

<sup>1</sup>Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit 1 - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

- A. If a component failed, use the component directly involved
- B. If not a component failure, use the related component e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or
explain the circumstances of the shutdown or power reduction.
The column should include the specific cause for each shutdown or significant power reduction and the immediate and
contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the
major safety-related corrective maintenance performed during
the outage or power reduction including an identification of
the critical path activity and a report of any single release of
radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent
of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

## REFUELING INFORMATION

eduled date for next refueling shutdown. 1/1/83  eduled date for restart following refueling.3/15/83  I refueling or resumption of operation thereafter require a finical specification change or other license amendment? answer is yes, what, in general, will these be? answer is no, has the reload fuel design and core configuration reviewed by your Plant Safety Review Committee to determine ther any unreviewed safety questions are associated with the ereload (Ref. 10 CFR Section 50.59)?  Es. Reload report and associated proposed Specification change or different fuel design or supplier, unreviewed design or formance analysis methods, significant changes in fuel design, operating procedures.  Il reload 72 fresh fuel assemblies and operate for approximate
I refueling or resumption of operation thereafter require a minical specification change or other license amendment? answer is yes, what, in general, will these be? answer is no, has the reload fuel design and core configuration reviewed by your Plant Safety Review Committee to determine ther any unreviewed safety questions are associated with the reload (Ref. 10 CFR Section 50.59)?  The second report and associated proposed Specification change and information. 10/1/82  The proposed specification and corting information. 10/1/82  The proposed specification and corting information considerations associated with refueling, e.g. or different fuel design or supplier, unreviewed design or formance analysis methods, significant changes in fuel design, operating procedures.
Annical specification change or other license amendment?  Answer is yes, what, in general, will these be?  Answer is no, has the reload fuel design and core configuration reviewed by your Plant Safety Review Committee to determine ther any unreviewed safety questions are associated with the reload (Ref. 10 CFR Section 50.59)?  Ass. Reload report and associated proposed Specification change and proposed specification change or the second section of the second
eduled date(s) for submitting proposed licensing action and corting information. 10/1/82  ortant licensing considerations associated with refueling, e.g. or different fuel design or supplier, unreviewed design or formance analysis methods, significant changes in fuel design, operating procedures.
orting information. 10/1/82  ortant licensing considerations associated with refueling, e.g. or different fuel design or supplier, unreviewed design or formance analysis methods, significant changes in fuel design, operating procedures.
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or different fuel design or supplier, unreviewed design or formance analysis methods, significant changes in fuel design, operating procedures.
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months. Four of which will be high burn-up test assemblies.
were the transfer of the trans
number of fuel assemblies (a) in the core and (b) in the spent storage pool. a) 177 b) 244
present licensed spent fuel pool storage capacity and the size ny increase in licensed storage capacity that has been request s planned, in number of fuel assemblies.
ent 589 increase size by 0
projected date of the last refueling that can be discharged he spent fuel pool assuming the present licensed capacity.
: 1986
he s

## NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY - DECEMBER, 1981

### UNIT 1

Unit One began the month at 85.97% full power. The unit is in a power limited condition due to fouling of the "A" Steam Generator. Periodic power adjustments have been required throughout the month to prevent flooding of the feedwater nozzles. Unit One ended the month at 88.30% full power with further power adjustments anticipated.