



Public Service Electric and Gas Company P.O. Box 236 Hancock Bridge, New Jersey 08038
Hope Creek Generating Station

January 14, 1991

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for December are being forwarded to you with the summary of changes, tests, and experiments for December 1990 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

J. J. Hagan
General Manager -
Hope Creek Operations

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Attachments

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The Energy People

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INDEX

<u>SECTION</u>	<u>NUMBER OF PAGES</u>
Average Daily Unit Power Level.	1
Operating Data Report	2
Refueling Information	1
Monthly Operating Summary	1
Summary of Changes, Tests, and Experiments.	7

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 1/14/91
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH December 1990

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. 995
2. 1075
3. 1064
4. 1048
5. 1064
6. 1083
7. 1035
8. 1009
9. 1014
10. 960
11. 1027
12. 1030
13. 1044
14. 1051
15. 1015
16. 1064

17. 996
18. 1087
19. 1027
20. 1110
21. 1071
22. 1044
23. 1018
24. 1067
25. 916
26. 0
27. 0
28. 0
29. 0
30. 0
31. 0

OPERATING DATA REPORT

DOCKET NO. 50-354
 UNIT Hope Creek
 DATE 1/14/91
 COMPLETED BY V. Zabielski
 TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period December 1990 Gross Hours in Report Period 744

2. Currently Authorized Power Level (MWt) 3293
 Max. Depend. Capacity (MWe-Net) 1031
 Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>
5. No. of hours reactor was critical	<u>602.0</u>	<u>8020.0</u>	<u>29,781.5</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>601.0</u>	<u>7941.9</u>	<u>29,293.1</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>1,213,429</u>	<u>25,586,032</u>	<u>92,542,408</u>
10. Gross electrical energy generated (MWH)	<u>649,080</u>	<u>8,465,410</u>	<u>30,621,673</u>
11. Net electrical energy generated (MWH)	<u>619,598</u>	<u>8,100,135</u>	<u>29,256,684</u>
12. Reactor service factor	<u>80.9</u>	<u>91.6</u>	<u>84.2</u>
13. Reactor availability factor	<u>80.9</u>	<u>91.6</u>	<u>84.2</u>
14. Unit service factor	<u>80.8</u>	<u>90.7</u>	<u>82.9</u>
15. Unit availability factor	<u>80.8</u>	<u>90.7</u>	<u>82.9</u>
16. Unit capacity factor (using MDC)	<u>80.8</u>	<u>89.7</u>	<u>80.3</u>
17. Unit capacity factor (Using Design MWe)	<u>78.0</u>	<u>86.7</u>	<u>77.6</u>
18. Unit forced outage rate	<u>0.0</u>	<u>5.2</u>	<u>5.5</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration): None			
20. If shutdown at end of report period, estimated date of start-up: 2/13/91			

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354
UNIT Hope Creek
DATE 1/14/91
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH December 1990

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
13	12/26	S	143	C	1	Refuel

Summary

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

DECEMBER 1990

Hope Creek entered the month of December at approximately 100% power. On December 26th, the unit was manually shutdown after completing 36 days of continuous power operation. The third refueling outage is currently underway with startup scheduled for February 13, 1991.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

DECEMBER 1990

The following Design Change Packages (DCP's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The DCP's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The DCP's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCPDescription of Design Change Package

- 4EC-3149 This DCP added rigging attachment points to allow for easier removal and replacement of valves and actuators. The valves and actuators addressed in this DCP are in the Fuel Pool Cooling, Torus Water Cleanup, Safety Auxiliaries Cooling, Instrument Gas, and Breathing Air Systems.
- 4EC-3187 This DCP added three pipe supports and modified two existing pipe supports on the outer radius instrumentation lines attached to the Reactor Recirculation System flow elbows. This DCP will improve the design by reducing the potential for fatigue induced failure due to vibration.
- 4HC-0212/01 This DCP upgraded the "A" Chilled Water Pump by replacing the motor and impeller. This upgrade will allow the Chilled Water System to operate with 2 pumps running and the other in standby.
- 4HC-0224 This DCP replaced the early warning fire detector in the Service Water Intake Structure with a Condensation Nuclei Fire Detector. The original fire detection system was not able to withstand the harsh atmosphere of the Intake Structure. The new system is insensitive to dust, dirt, wind, vibration, humidity, temperature, and electronic noise.
- 4HC-0245/02 This DCP added new flow indicators and replaced existing pressure indicators in the "C" and "D" Diesel Generator Fuel Oil Transfer Storage Tanks. This will eliminate the need to use portable Measurement and Test Equipment during the performance of the Diesel Fuel Oil Transfer Pump Surveillance Test.
- 4HC-0245/03 This DCP added new flow indicators and replaced existing pressure indicators in the "E" and "F" Diesel Generator Fuel Oil Transfer Storage Tanks. This will eliminate the need to use portable Measurement and Test Equipment during the performance of the Diesel Fuel Oil Transfer Pump Surveillance Test.
- 4HC-0245/04 This DCP added new flow indicators and replaced existing pressure indicators in the "G" and "H" Diesel Generator Fuel Oil Transfer Storage Tanks. This will eliminate the need to use portable Measurement and Test Equipment during the performance of the Diesel Fuel Oil Transfer Pump Surveillance Test.

The following Temporary Modification Requests (TMR's) have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The TMR's did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. The TMR's did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

TMRDescription of Temporary Modification Request

- 90-074 This TMR installed a jumper to disable an input to an overhead annunciator. This was a nuisance alarm due to a failed Trip Coil Continuity Alarm Card. The TMR will be removed when a new card is available for installation.
- 90-075 This TMR removed the Reactor Vessel Water Level Transmitter and installed a pressure transmitter in its place. It also rescaled the Reactor Level Shutdown Range Indicator. These modifications will provide the Control Room with reactor level indication during floodup.
- 90-076 This TMR installed an air compressor connection in the discharge line of a Service Air Compressor Aftercooler. This connection will provide an air supply for the Service Air System while the normal air compressors are out of service for system outages.

The following Deficiency Report (DR) has been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The DR did not create a new safety hazard to the plant nor did it affect the safe shutdown of the reactor. The DR did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluation determined that no unreviewed safety or environmental questions are involved.

DR

Description of Deficiency Report

HTE-90-0111

This DR addresses a pinhole leak in the Station Service Water System. The leak is in the "A" Safety Auxiliaries Cooling System Heat Exchanger Room, on the "A" Service Water Supply Line to the "A" Safety Auxiliaries Cooling System Heat Exchangers. The DR authorizes the pipe to be "used-as-is" until the completion of the "B" Loop Service Water piping replacement work that is being conducted during the third refueling outage.