CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

January 15, 1991
Re: Technical Specification 6.9.1.8
Docket No. 50-213

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1.8, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 90-12 covering operations for the period December 1, 1990 to December 31, 1990 is hereby forwarded.

Very truly yours,

John P. Stetz
Station Director

JPS/va

cc: (1) Regional Administrator, Region 1
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

(2) John T. Shedlosky
Sr. Resident Inspector
Connecticut Yankee

IE24
1/1

Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 90-12

For The Month of

December 1990

Plant Operations Summary - December 1990

The following is a summary of Plant Operations for December 1990.

On December 1st at 0000 hours, the plant was operating at 100% rated power.

On December 12th at 1102 hours, the #3 steam generator feed regulator valve began acting erratically. The valve was placed in manual control and a power reduction was initiated. At 1145 hours, the steam generator level had stabilized, and the power reduction was halted at 96%. By 1930 hours, the #3 steam generator feed regulator valve was repaired and a power ascension was commenced.

On December 13th at 0926 hours, the plant was operating a 100% power.

On December 22nd at 2300 hours, a power reduction was initiated for the turbine stop and control valve test.

On December 23rd at 0020 hours, power was at 65%. At 0140 hours, the test was successfully completed. At 0210 hours, a power ascension was commenced. By 0811 hours, the plant was operating at 100% power.

On December 24th at 0238 hours, system dispatcher, due to system loads, requested a 21 Mwe load reduction. At 0250 hours, a power reduction was initiated. By 0300 hours, power was stabilized at 96%. At 0507 hours the dispatcher requested a return to full power. At 0510 hours, a power ascension was commenced. By 0635 hours, the plant was operating at 100% power.

On December 27th at 1335 hours, river debris caused high differential pressures on the Service Water filters. A power reduction was initiated and the "B" filter was removed from service for cleaning. At 1447 hours, the "B" filter was returned to service and the power reduction was halted at 70%. At 2115 hours, the "A" filter was removed from service for cleaning.

On December 27th at 2311 hours the "A filter was returned to service. On December 28th at 0820 a power ascension was commenced. By 1341 hours, the plant was operating at 100% power.

The plant continued to operate at 100% power for the remainder of the month.

I & C

System or Component	Malfunction		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide for Reactor Safety During Repair
	Cause	Result			
RCS Flow Transmitter (Loop 1)	Improperly assembled tubing union	Leak, increase in airborne activity	None	Inspecting a sample of other similar unions	Load reduction to place the plant in a mode in which the low flow Rx trip is not required

MAINTENANCE

System or Component	Malfunction		Effect on Safe Operation	Corrective Action Taken to Prevent Repetition	Special Precautions Taken To Provide for Reactor Safety During Repair
	Cause	Result			
FL-53-1A & B Service Water Adams Filters	Excessive dirt in river	Filter elements plugged	None	Increased Operations Surveillance	Power reduction, standby filter left in backwash

AVERAGE DAILY UNIT POWER LEVEL

Docket No: 50-213

Unit: Connecticut Yankee
Haddam Neck

Date: December 1990

Completed By: S. F. Claffey
Telephone: (203) 267-3650

Month: December

DAY	AVERAGE POWER LEVEL (MWe-Net)	DAY	AVERAGE POWER LEVEL (MWe-Net)
1	587	17	588
2	587	18	588
3	587	19	587
4	587	20	587
5	588	21	588
6	588	22	584
7	588	23	537
8	587	24	586
9	588	25	589
10	587	26	589
11	588	27	522
12	578	28	520
13	587	29	588
14	587	30	589
15	588	31	588
16	588		

CONNECTICUT YANKEE
REACTOR COOLANT DATA
MONTH: DEC 90

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
PH @ 25 DEGREES C	6.09E+00	6.46E+00	6.57E+00
CONDUCTIVITY (UMHOS/CM)	1.38E+01	1.73E+01	2.16E+01
CHLORIDES (PPM)	<5.00E-02	<5.00E-02	<5.00E-02
DISSOLVED OXYGEN (PPB)	<5.00E+00	<5.00E+00	<5.00E+00
BORON (PPM)	7.97E+02	8.51E+02	8.91E+02
LITHIUM (PPM)	1.23E+00	1.60E+00	2.14E+00
TOTAL GAMMA ACT. (UC/ML)	4.40E-01	8.06E-01	1.37E+00
IODINE-131 ACT. (UC/ML)	4.65E-03	6.07E-03	9.66E-03
I-131/I-133 RATIO	1.26E+00	1.66E+00	2.53E+00
CRUD (MG/LITER)	<1.00E-02	5.22E-02	1.20E+00
TRITIUM (UC/ML)	9.71E-01	1.42E+00	1.75E+00
HYDROGEN (CC/KG)	2.34E+01	2.53E+01	2.65E+01

AERATED LIQUID WASTE PROCESSED(GALLONS):	1.68E+05
WASTE LIQUID PROCESSED THROUGH BORON RECOVERY(GALLONS):	5.60E+04
AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE):	6.27E-01
PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE):	1.36E-03

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 12/90 Outage + On-line Hours: 0.0 + 744.0 = 744.0
3. Utility Contact: J. Stanford (203) 267-3635
4. Licensed Thermal Power (MWt): 1825
5. Nameplate Rating (Gross MWe): 667 x 0.9 = 600.3
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 591.8
8. Maximum Dependable Capacity (Net MWe): 565
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	744.0	8,760.0	201,624.0
13. Hours reactor critical:	744.0	2,824.5	161,073.7
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	744.0	2,589.9	154,677.6
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (MWtH):	1,338,960.0	3,805,695.0	266,673,013.0 *
18. Gross electrical energy generated (MWeH):	452,813.0	1,249,792.0	87,342,838.0 *
19. Net electrical energy generated (MWeH):	432,512.4	1,153,918.6	82,976,898.6 *
20. Unit service factor:	100.0	29.6	76.7
21. Unit availability factor:	100.0	29.6	76.9
22. Unit capacity factor using MDC net:	102.9	23.3	74.9
23. Unit capacity factor using DER net:	99.9	22.6	70.7
24. Unit forced outage rate:	0.0	22.1	5.9
25. Forced outage hours:	0.0	734.7	9,710.3

26. Shutdowns scheduled over next 6 months (type,date, duration): NONE

27. If currently shutdown, estimated startup date: N/A

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

UNIT SHUTDOWN AND POWER REDUCTIONS

Docket No. 50-213
Unit Name: Connecticut Yankee
Date: December 1990
Completed By: S. Claffey
Telephone 203-267-3650

Report Month : December

No.	Date	1: Type	Duration (hours)	2: Reason	3: Method of Shutting Down Reactor	LER Report	4 System Code	Component Code	Cause and Corrective Action to Prevent Recurrence
90-08	12/27/90	F	24.1	B	N/A	N/A	BI	FLT	Commenced Load reduction due to plugging of service water (Adams) filters

1
F Forced
S Scheduled

2
Reason
A Equipment Failure (explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training & License Examination
F Administrative
G Operational Error (Explain)
H Other (Explain)

3
Method
1 Manual
2 Manual Scram
3 Automatic Scram
4 Other (Explain)

4
Exhibit G - Instructions for
Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 Same Source

Refueling Information Request

1. Name of facility

   Haddam Neck

2. Scheduled date for next refueling shutdown.

   October 5, 1991

3. Scheduled date for restart following refueling.

   November 26, 1991

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   Yes

   (b) If answer is yes, what, in general, will these be?

   Revise Section 5 of Technical Specifications to allow use of zircaloy clad fuel. Obtain an exemption from 10CFR50 Appendix K Sections I.D.3, I.D.4 and I.D.5.

   (c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?
   (Ref. 10 CFR section 50.59)

   n/a

   (d) If no such review has taken place, when is it scheduled?

   n/a

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   The exemption request was submitted to the NRC in September 1990. The request for license amendment will be submitted in March 1991.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Conversion to zircaloy cladding.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   (a) 157 (b) 709

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   1168

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   1996