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January 14, 1991
ND3MNO:3088

Beaver Valley Power Station, Unit No. 2
Docket No. 50-412, License No. NPF-73
LER 90-027-00

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 90-027-00, 10 CFR 50.73.a.2.ii, "Excessive Airflow through Control Room Outside Air Inlet Damper".

Very truly yours,

K.L. Ostrowski for

T. P. Noonan
General Manager
Nuclear Operations

JGT/sl

Attachment

IE22
1/1

cc: Mr. T. T. Martin, Regional Administrator
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NRC FORM 366 (6-89)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104
EXPIRES: 4/30/92

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Beaver Valley Power Station Unit 2	0 5 0 0 0 4 1 2	1 OF 0 3

TITLE (4)

Excessive Airflow through Control Room Outside Air Inlet Damper

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									N/A	0 5 0 0 0
1 2	1 3	9 0	9 0	— 0 2 7	— 0 0	0 1	1 4	9 1		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
T.P. Noonan, General Manager Nuclear Operations	4 1 2 6 4 3 - 1 2 5 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	V I	D M P	A 3 4 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE)	NO		0 6	0 1	9 1

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 12/13/90, with the Unit in Power Operation at 100% reactor power, a control room ventilation balancing procedure was being performed. On 12/14/90 at 1040 hours, the control room outside air intake damper was found to allow 600 cubic feet per minute (cfm) air flow. The designed maximum air flow is 200 cfm. This value is used to support the analyses for control room personnel radiological dose projection in the accident analyses. The control room ventilation was placed on recirculation. A maintenance work request was generated to inspect the damper. The damper was found to have degraded gasket sealing material. The gasket material was replaced. An evaluation of the projected dose estimates to control room personnel, based on the as-found air flow rates, will be performed.

DESCRIPTION OF EVENT

On 12/13/90, with the Unit in Power Operation at 100% reactor power, a control room ventilation balancing procedure was being performed. On 12/14/90 at 1040 hours, the control room outside air intake damper, 2HVC-DMP215, was found to allow 600 cubic feet per minute (cfm) air flow. The designed maximum intake air flow is 200 cfm. This value is used to support the analyses for control room personnel radiological dose projection in the accident analyses. The Unit 2 portion of the control room ventilation was placed on full recirculation. Maintenance personnel were requested to investigate and repair the damper.

CAUSE OF THE EVENT

The cause of the event was degraded gasket sealing material. This condition allowed greater than design air flow in the fully closed position.

CORRECTIVE ACTIONS

The following corrective actions have been or will be taken as a result of this event:

1. The Unit 2 control room was placed on full recirculation (outside air was isolated).

2. The Unit 1 air intake was measured and found to be within its 300 cfm maximum limit.

3. A maintenance work request was generated for repair of the damper. The gasket sealing material was replaced on 12/17/90.

4. The control room ventilation balancing procedure was performed following damper repair and the outside air damper was positioned to allow 178 cfm air flow.

5. An evaluation will be performed to determine the effects of the excess air flow on the accident analyses dose estimates for the time period of degraded damper operation.

REPORTABILITY

This event was reported to the Nuclear Regulatory Commission in accordance with 10CFR50.72.b.1.ii.B, at 1133 hours on 12/14/90. This written report is being submitted in accordance with 10CFR50.73.a.2.ii.B, as a condition that was potentially outside the design basis of the accident analyses.

SAFETY IMPLICATIONS

The Safety Analysis assumes a 500 cfm maximum outside air flow to the defined control room envelope. A maximum of 200 cfm of this is the assumed contribution from the unit 2 ventilation system. The balance of the intake is assumed to come from the Unit 1 ventilation system. An evaluation will be performed to determine the extent of change to the existing dose projections for control room personnel during the time frame in which degraded damper operation was present. A supplemental report will be issued describing the resolution of the evaluation being performed by June 1, 1991.