



**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
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Telephone 815/357-6761

January 10, 1991

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for December, 1990.

Very truly yours,

for G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

11. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, generator on-line at 975 MWE Reducing power due to rod 50-47 drift.
	0230	Reduced power to 750 MWE to pull rod 50-47.
	0800	Increased power to 975 MWE.
1	2300	Increased power to 1130 MWE.
4	0000	Reduced power to 1000 MWE for CRD exercising.
	1100	Increased power to 1130 MWE.
11	0100	Reduced power to 850 MWE for rod set.
	1200	Increased power to 1130 MWE.
25	1500	Reduced power to 1000 MWE for system load.
	1900	Increased power to 1104 MWE.
28	2100	Reduced power to 850 MWE to accomplish rod moves and to take the turbine driven reactor feed pump off line.
30	1100	Increased power to 1130 MWE.
31	2400	Reactor critical, generator on line at 1130 MWE.

- B. PLANT OR PROCEDURE CHANGES, TESTS, OR EXPERIMENTS (50.59) (Unit 1)
1. Amendments to the Facility License or Technical Specification.  
Primary Containment Penetration Overcurrent Protection
  2. Changes to procedures which are described in the Safety Analysis Report.
    - a) LaSalle Special Test LST-90-133, "Control Room HVAC System (VC) Air Flow Data Collection", was approved to reposition the balancing dampers on Control Room/Auxiliary Electric Equipment Room HVAC System. The purpose of this test is to take data on the VC System and input to engineering. This test consists of the data collection portion of approved LST-90-054. This test will only change the flowrate to ensure the control room meets the requirements of Technical Specification 4.7.2.d. The safety evaluation concluded that the temperature in the control room will be maintained between 55°F and 104°F to ensure that the equipment is maintained operable.
  3. Changes to facility which are described in the Safety Analysis Report.
    - a) LaSalle Temporary System Change (TSC) 1-879-90 was performed to bypass Division 1 125 VDC Battery Charger DC Voltage Lo/Failure Alarm due to chattering, causing a nuisance alarm. This alarm is discussed in UFSAR 8.3.2.1.1. Even though continuous indication of this alarm condition is lost during the time that it is bypassed, charger voltage indication is still available in the control room and recorded shiftly. The Safety Evaluation concluded that there are redundant alarms available (DC Bus Undervoltage) which would also alert operators to charger problems and lifting of this alarm lead in no way affects the ability of the charger to perform its design function.
  4. Tests and Experiments not described in the Safety Analysis Report.  
(None)
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)
- D. COMPLETED SAFETY-RELATED MODIFICATIONS.  
(None)

C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L04237	Main Steam Line High Differential Pressure Switch	Switch	Switch out of calibration	Replaced switch
L04271	RCIC Pump Suction Valve	Contactors	Erratic operation	Replaced contactors

(No SOR failures this month)

E. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
90-013-00	12-07-90	30 day missed surv. testing of RH IX Vent Valves due to procedure deficiency.
90-014-00	12-25-90	30 day loss of Unit 1 250 Volt Batteries/RCIC System.

F. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 3)
2. Average Daily Unit Power Level (See Table 4)
3. Unit Shutdowns and Significant Power Reductions (See Table 5)



TABLE 5  
 F.1 OPERATING DATA REPORT

DOCKET NO. 850-373  
 UNIT LADALLE ONE  
 DATE JANUARY 10, 1991  
 COMPLETED BY M.J. CIALKOWSKI  
 TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1990 GROSS HOURS IN REPORTING PERIOD: 744  
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3.323 MAX DEMAND CAPACITY (MWe-Net): 1,036  
 DESIGN ELECTRICAL RATING (MWe-Net): 1,070  
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (NONE)  
 4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	8,475.3	40,609.4
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	8,330.5	39,740.8
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWHt)	2,409,000	26,370,033.4	114,203,601
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	824,000	8,934,747	38,003,200
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	796,683	8,637,380	36,370,082
12. REACTOR SERVICE FACTOR (%)	100.0	96.8	66.1
13. REACTOR AVAILABILITY FACTOR (%)	100.0	96.8	66.8
14. UNIT SERVICE FACTOR (%)	100.0	95.1	64.7
15. UNIT AVAILABILITY FACTOR (%)	100.0	95.1	64.7
16. UNIT CAPACITY FACTOR (USING MDC) (%)	103.4	95.2	57.2
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	99.3	91.5	55.0
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	1.6	8.5

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
 Refueling (LIR04) 02/16/91 10 WEEKS

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:  
 (N/A)

TABLE A  
F.2 AVERAGE DAILY UNIT POWER LEVEL (NWe-Net)

DOCKET NO. 850-373  
UNIT LABALLE ONE  
DATE JANUARY 10, 1991  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: DECEMBER 1990

DAY	POWER	DAY	POWER
1	916	17	1,095
2	1,020	18	1,095
3	1,100	19	1,088
4	1,001	20	1,078
5	1,099	21	1,079
6	1,099	22	1,073
7	1,096	23	1,071
8	1,096	24	1,071
9	1,096	25	1,057
10	1,084	26	1,074
11	1,026	27	1,070
12	1,099	28	1,028
13	1,097	29	929
14	1,097	30	1,007
15	1,091	31	1,109
16	1,096		

TABLE 5

F.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(Unit 1)

YEARLY SEQUENTIAL DATE NUMBER	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
(None)					

## SUMMARY OF OPERATION:

The Unit remained on line at high power throughout the month. Several minor power reductions were required due to low grid demand, routine surveillances, and Rod 50-47 Drift.

G. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT
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(None)

2. ECCS System Outages  
(See Table 6)

3. Changes to the Off-Site Dose Calculation Manual  
(None)

4. Major changes to Radioactive Waste Treatment Systems.  
(None)

5. Indications of Failed Fuel Elements.  
(None)

(Unit 1)  
Table 6

G.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
420	ODG023B	Inspection of check valve.
424	ODG023A	Inspection of check valve.
425	ODG01K	Routine calibration and surveillances
(U-1)		
687	1DG01K	Lubrication.
694	1E12-F047A	Administrative control.
695	1E12-F047A	Perform surveillance LES-EQ-112.
701	1E51-F064	Administrative control.
706	1E12-F087A	Perform surveillance LES-EQ-112.
714	1E51-C003	Perform inspection.
715	1E51-F010	Perform surveillance LES-EQ-112.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

DECEMBER 1990

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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(Unit 2)

I. INTRODUCTION

II. REPORT

- A. SUMMARY OF OPERATING EXPERIENCE
- B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE (50.59)
  - 1. Amendments to Facility License or Technical Specifications
  - 2. Changes to procedures which are described in the Safety Analysis Report.
  - 3. Changes to facility which are described in the Safety Analysis Report.
  - 4. Tests and Experiments not covered in the Safety Analysis Report.
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT
- D. COMPLETED SAFETY-RELATED MODIFICATIONS
- E. LICENSEE EVENT REPORTS
- F. DATA TABULATIONS
  - 1. Operating Data Report
  - 2. Average Daily Unit Power Level
  - 3. Unit Shutdowns and Power Reductions
- G. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.



II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, generator on line at 1135 MWE
4	2330	Reduced power to 1070 MWE for CRD exercising
5	1000	Increased power to 1135 MWE
7	2100	Reduced power to 850 MWE for lubrication of the "B" Turbine Driven Reactor Feed Pump and to place the Motor Driven Reactor Feed Pump on line
8	0600	Increased power to 1000 MWE
10	0200	Increased power to 1135 MWE
12	0300	Reduced power to 1050 MWE for CRD exercising and insertion of FCL rods
	1000	Increased power to 1135 MWE
13	1130	Reduced power to 250 MWE due to leakage in the "23C" Low Pressure Feedwater Heater
15	0155	Generator off line
	1300	Reactor subcritical
19	1300	Reactor achieves criticality
20	1819	Generator on line
21	2300	Increased power to 500 MWE
22	0600	Increased power to 1013 MWE
	0900	Reduced load to 900 MWE due to "A" Turbine Driven Reactor Feed Pump Lube Oil Pump fire
	1500	Increased power to 1013 MWE after transfer of the "A" Turbine Driven Reactor Feed Pump with the Motor Driven Reactor Feed Pump
26	0100	Reduced power to 800 MWE for weekly surveillances, monthly surveillances and for final rod set
	2100	Increased power to 1120 MWE
	2300	Reduced power to 1000 MWE due to a high level alarm from the "21A" Low Pressure Feedwater Heater
30	0200	Increased power to 1130 MWE
31	2400	Reactor critical, generator on line at 1130 MWE

- B. PLANT OR PROCEDURE CHANGES, TESTS OR EXPERIMENTS (50.59) (Unit 2)
1. Amendments to the Facility License or Technical Specification.  
Primary Containment Penetration Overcurrent Protection.
  2. Changes to procedures which are described in the Safety Analysis Report.  
(None.)
  3. Changes to facility which are described in the Safety Analysis Report.  
(None)
  4. Tests and Experiments not described in the Safety Analysis Report.  
(None)
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SCF differential pressure switch failure reports).  
(See Table 1)
- D. COMPLETED SAFETY RELATED MODIFICATIONS.  
(None)
- E. LICENSEE EVENT REPORTS (Unit 2)
- | <u>LER Number</u> | <u>Date</u> | <u>Description</u>                                     |
|-------------------|-------------|--|
| 90-012-00         | 12/15/90    | 30 day missed tech spec on H2 Analyzers                |
| 90-013-00         | 12/17/90    | 30 day MSIV isolation during performance of LIS-MS-406 |
| 90-C14-00         | 12/22/90    | 30 day 2A Turbine Driven Reactor Feed Pump fire        |
- F. DATA TABULATIONS (Unit 2)
1. Operating Data Report.  
(See Table 3)
  2. Average Daily Unit Power Level.  
(See Table 4)
  3. Unit Shutdowns and Significant Power Reductions.  
(See Table 5)

C TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L00567	Hydraulic Control Unit 34-39	Filter leaking	None	Replace filter
L02677	Hydraulic Control Unit 46-07	Withdrawal valve 122	Degraded control rod operation	Rebuilt withdrawal valve 122
L04502	LPRM 40-09B	Power supply	None	Replaced power supply

(No SOR failures this month)

TABLE 3  
F.1 OPERATING DATA REPORT

DOCKET NO. 050-374  
UNIT LASALLE TWO  
DATE JANUARY 10, 1991  
COMPLETED BY H.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD	DECEMBER 1990	GROSS HOURS IN REPORTING PERIOD	744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt)	3,323	MAX DEPEND CAPACITY (MWe-Net)	1,636
		DESIGN ELECTRICAL RATING (MWe-Net)	1,678
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net)	(NONE)		
4. REASONS FOR RESTRICTION (IF ANY)			

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	648.9	6,343.1	36,490.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	667.5	6,164.1	35,647.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.6	0.0	0.0
9. THERMAL ENERGY GENERATED (MMBtu)	1,760,166	19,119,127.2	105,439,175
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	595,234	6,432,506	34,893,832
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	573,540	6,183,785	33,419,090
12. REACTOR SERVICE FACTOR (%)	87.1	72.4	67.1
13. REACTOR AVAILABILITY FACTOR (%)	87.1	72.4	70.3
14. UNIT SERVICE FACTOR (%)	81.7	70.4	65.9
15. UNIT AVAILABILITY FACTOR (%)	81.7	70.4	65.9
16. UNIT CAPACITY FACTOR (USING BDC) (%)	74.4	68.1	59.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	71.5	65.5	57.0
18. UNIT FORCED OUTAGE FACTOR (%)	16.3	7.2	14.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)  
(NONE)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP  
(N/A)

TABLE 4  
 F.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374  
 UNIT LASALLE TWO  
 DATE JANUARY 10, 1991  
 COMPLETED BY M.J. CIALKOWSKI  
 TELEPHONE (815)-357-6761

REPORT PERIOD: DECEMBER 1990

DAY	POWER	DAY	POWER
1	1,097	17	-12
2	1,099	18	-12
3	1,101	19	-12
4	1,190	20	21
5	1,085	21	349
6	1,100	22	891
7	1,057	23	951
8	949	24	953
9	1,001	25	954
10	1,098	26	961
11	1,099	27	965
12	1,005	28	974
13	552	29	1,012
14	300	30	1,094
15	-7	31	1,094
16	-12		

TABLE 5

F.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%  
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
14	901213	F	0.0	A	4	Power level reduced due to major tube leaks in the 23C Low Pressure Heater
15	901214	F	0.0	A	4	Continuation of power level reduction due to tube leakage
16	901215	F	136.5	A	1	Repair of tube leaks in the 23C Low Pressure Heater

## SUMMARY OF OPERATION:

The unit remained on line at high power throughout most the month. Several minor power reductions were required due to routine surveillances and turbine driven reactor feed pump work. The unit experienced a forced outage due to leakage in the "23C" Low Pressure Feedwater Heater. The outage started on 12/15/90 and the unit was returned to service on 12/20/90.



G. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief		Valve	Operations	
DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT
(None)				
	2. ECCS System Outages (See Table 6)			
	3. Changes to the Off-Site Dose Calculation Manual. (None)			
	4. Major changes to Radioactive Waste Treatment Systems. (None)			
	5. Indications of Failed Fuel Elements. (None)			

## G.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1804	2E51-C004	Inspection per Procedure LES-DC-104
1805	2E51-C005	Inspection per Procedure LES-DC-104 Replaced motor heater
1807	2E51-F360	Replaced motor in limitorque
1809	2E51-C002	Lubricate and change oil
1820	2E51-F064 2E51-F091	For completion of on-site review #90-043
1887	2E22-S001	Perform calibrations