VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT NO. 82-09

SEPTEMBER, 1982

APPROVED BY: _____
Station Manager

OPERATING DATA REPORT

DOCKET NO. 50-280
DATE 04 OCT 82
COMPLETED BY VIVIAN H. JONES
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1	
2. REPORTING PERIOD	90182 TO 93082	
3. LICENSED THERMAL POWER (MWT)	2441	NOTES
4. NAMEPLATE RATING (GROSS MWE)	847.5	
5. DESIGN ELECTRICAL RATING (NET MWE)	788	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775	
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A	
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A	
10. REASONS FOR RESTRICTIONS, IF ANY	N/A	

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	6551.0	85679.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	6088.2	52122.7
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3731.5
14. HOURS GENERATOR ON-LINE	720.0	6016.9	51091.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	3736.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1746999.8	14329586.5	118657852.9
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	545315.0	4498615.0	38317828.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	516139.0	4262702.0	36339938.0
19. UNIT SERVICE FACTOR	100.0 °/°	91.8 °/°	59.6 °/°
20. UNIT AVAILABILITY FACTOR	100.0 °/°	91.8 °/°	64.0 °/°
21. UNIT CAPACITY FACTOR (USING MDC NET)	92.5 °/°	84.0 °/°	54.7 °/°
22. UNIT CAPACITY FACTOR (USING DER NET)	91.0 °/°	82.6 °/°	53.8 °/°
23. UNIT FORCED OUTAGE RATE	0.0	3.5 °/°	23.4 °/°
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	2/1/83 - 74 days - Refueling/10 yr. ISI		

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

OPERATING DATA REPORT

DOCKET NO. 50-281
DATE 04 OCT 82
COMPLETED BY VIVIAN H. JONES
TELEPHONE 804-357-3184

OPERATING STATUS

1.	UNIT NAME	SURRY UNIT 2
2.	REPORTING PERIOD	90182 TO 93082
3.	LICENSED THERMAL POWER (MWT)	2441
4.	NAMEPLATE RATING (GROSS MWE)	847.5
5.	DESIGN ELECTRICAL RATING (NET MWE)	788
6.	MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811
7.	MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775
8.	IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A
10.	REASONS FOR RESTRICTIONS, IF ANY	N/A

NOTES

		THIS MONTH	YR-TO-DATE	CUMULATIVE
11.	HOURS IN REPORTING PERIOD	720.0	6551.0	82559.0
12.	NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	5996.1	50856.8
13.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14.	HOURS GENERATOR ON-LINE	720.0	5925.2	50086.9
15.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16.	GROSS THERMAL ENERGY GENERATED (MWH)	1740572.6	13593469.1	116890755.0
17.	GROSS ELECTRICAL ENERGY GENERATED (MWH)	568135.0	4375525.0	38052704.0
18.	NET ELECTRICAL ENERGY GENERATED (MWH)	538620.0	4135015.0	36063730.0
19.	UNIT SERVICE FACTOR	100.0 °/°	90.4 °/°	60.6 °/°
20.	UNIT AVAILABILITY FACTOR	100.0 °/°	90.4 °/°	60.6 °/°
21.	UNIT CAPACITY FACTOR (USING MDC NET)	96.5 °/°	81.4 °/°	56.4 °/°
22.	UNIT CAPACITY FACTOR (USING DER NET)	94.9 °/°	80.1 °/°	55.4 °/°
23.	UNIT FORCED OUTAGE RATE	0.0	4.9 °/°	28.6 °/°
24.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE,DATE,AND DURATION OF EACH)	FALL MAINTENANCE-11-12-82-10 DAYS		

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-280
UNIT NAME Surry 1
DATE 10-07-82
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184x477

REPORT MONTH Sept. 1982

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
		NONE DURING THIS REPORTING PERIOD.							

[1] F: Forced
S: Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

[4] Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH Sept. 1982

DOCKET NO. 50-281
UNIT NAME Surry 2
DATE 10-07-82
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 x477

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-34	9-5-82	S	0.0	H	4				Reduced power to load follow on orders of the System Operator
82-35	9-6-82	S	0.0	H	4				Reduced power to load follow on orders of the System Operator.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 1 MONTH: Sept., 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON
			NONE DURING THIS REPORTING PERIOD.			
				MONTHLY TOTAL		

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2 MONTH: Sept., 1982

DATE	TIME	HOURS	LOAD, MW	REDUCTIONS, MW	MWH	REASON
			NONE DURING THIS REPORTING PERIOD.			
				MONTHLY TOTAL		

DOCKET NO 50-280
UNIT SURRY 1
DATE 10-1-82
COMPLETED BY VIVIAN H. JONES

AVERAGE DAILY UNIT POWER LEVEL

MONTH: SEPTEMBER 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	711.0	16	706.5
2	714.4	17	719.4
3	716.8	18	716.9
4	716.5	19	718.2
5	720.0	20	720.2
6	722.0	21	720.3
7	723.8	22	715.0
8	724.8	23	715.2
9	723.2	24	716.3
10	722.3	25	716.4
11	720.8	26	717.5
12	718.7	27	716.1
13	713.5	28	714.1
14	706.0	29	713.9
15	707.3	30	718.9

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

DOCKET NO 50-281
UNIT SURRY II
DATE 10-1-82
COMPLETED BY VIVIAN H. JONES

AVERAGE DAILY UNIT POWER LEVEL

MONTH: SEPTEMBER 82

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	751.6	16	750.7
2	751.3	17	752.4
3	749.5	18	751.0
4	744.7	19	749.6
5	693.9	20	749.5
6	681.8	21	747.1
7	755.7	22	753.4
8	754.9	23	756.0
9	757.7	24	755.3
10	757.0	25	755.1
11	754.0	26	[illegible]
12	752.7	27	756.8
13	750.0	28	751.2
14	745.5	29	755.9
15	748.1	30	754.3

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 o/o LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

SUMMARY OF OPERATING EXPERIENCE

September, 1982

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

September 1	This reporting period begins with unit at 100% power.
September 12	2319 - Turbine runback to 735 MWe/97% power. The runback was caused by a spike on Channel III over-temperature ΔT Protection with Channel I in trip due to a problem with excore detector N-41.
	2325 - Started increasing power to 100% - at 3% per hour.
September 13	0030 - The unit reached 100% power.
September 17	0745 - Chemist reported feedwater pH of 7.5. Commenced reducing power in preparation for going to Hot Shutdown in accordance with Abnormal Procedure 33 due to low pH of the feedwater.
	0754 - Chemist reported feedwater pH of 8.1. Stopped ramp and returned unit to 100% power. The cause of the low feedwater pH was found to be that the ammonia injection pump had tripped off.
September 30	This reporting period ends with the unit at 100% power.

UNIT TWO

September 1	This reporting period begins with the unit at 100% power.
September 5	0102 - Commenced reducing power to load follow on orders of the System Operator.
	0350 - Stopped reducing power at 525 MWe/65% power on orders of the System Operator.
	0703 - Started increasing power on orders of the System Operator.
	0900 - The unit reached 100% power.
September 6	0038 - Commenced reducing power to load follow on orders of the System Operator.

0340 - Stopped reducing power at 450 MWe/55% power on orders of the System Operator.

0549 - Started increasing power on orders of System Operator.

1130 - The unit reached 100% power.

September 30 This reporting period ends with the unit at 100% power.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

September, 1982

None during this reporting period.

FACILITY CHANGES REQUIRING NRC APPROVAL

September, 1982

None during this reporting period.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

September, 1982

		UNIT
D/C 80-22	Morpholine and Hydrozine Injection Into the Bearing Cooling System	1 & 2

This design change replaces the addition of chromates in the bearing cooling system with filming amines to comply with State Water Control Board discharge limitations to provide corrosion protection. This required a tap off of the present chemical feed pumps discharge with the line tied into the bearing cooling system.

SUMMARY OF SAFETY ANALYSIS

The modification does not affect the station or the operation of any safety related equipment.

D/C 80-51	Post-Accident Monitoring and Control Panel	1

This design change provides Post-Accident Monitoring Indication and Control in the control room as required by TMI-2, NREG-0578 and subsequent clarifications contained in the NRC letter dated October 30, 1979.

SUMMARY OF SAFETY ANALYSIS

This modification does not affect normal station operation of any safety related systems. The Technical Specifications and FSAR are not affected by this design change.

D/C 80-62	Storm Drain Radiation Sampling System	1 & 2

This design change added recording flow meters and automatic waste water samplers to four storm drain release points. The water is sampled to ensure that radioactive contaminants are not inadvertently discharged into the river.

SUMMARY OF SAFETY ANALYSIS

The addition does not affect the operation of any safety related equipment nor will it affect the implementation of any station procedures.

D/C 80-74	Manual Lineup of Air Ejector Vent	1 & 2

This design change eliminated the requirement of additional effluent monitors for the extended range capability. The condenser air ejector discharge lines shall be rerouted having new connections upstream of the Ventilation Vent Stack high range effluent monitor for use during accident conditions.

SUMMARY OF SAFETY ANALYSIS

The modification does not affect the operation of any safety-related equipment. It provides greater flexibility in system operation under high radiation or loss of normal effluent monitoring.

		UNIT
D/C 81-12	Liquid Waste Monitor RM-LW-108 Relocation	1 & 2

This design change relocated liquid waste monitor (RM-LW-108). The radiation monitor was giving false indications of radiation levels due to high background, internal contamination and crude build up.

SUMMARY OF SAFETY ANALYSIS

The modification will improve the performance of liquid waste disposal system. Relocation of piping to allow monitoring of the effluents from a low backgroud area will not violate the intent of the Technical Specifications nor the FSAR.

D/C 81-19	Machine Shop Replacement Facility	1 & 2

D/C 81-19G Machine Shop Relocation Facility-Sprinkler System portion was implemented.

SUMMARY OF SAFETY ANALYSIS

The addition of the MSRF does not minimize the safety of operating units or effect the operation of safety-related equipment.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

September, 1982

None during this reporting period.

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

September, 1982

Special Test No.	Title	Unit	Date Completed
ST-52	RCS Flow Measurement Data	1, 2	09-16-82
ST-145	Auxiliary Building Vent - Vent Delay Time Test	1, 2	09-30-82

OTHER CHANGES, TESTS AND EXPERIMENTS

September, 1982

None during this reporting period.

SURRY POWER STATION CORRECTED COPY

CHEMISTRY REPORT

August 19 82

T.S. 6.6.3.d

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., µCi/ml	(A) 5.73 Eo	1.74 Eo	3.13 Eo	8.60 E-1	1.62 E-1	2.39 E-1
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, µCi/ml	1.34 E-1	6.10 E-2	1.08 E-1	3.38 E-1	2.50 E-1	3.03 E-1
Iodine-131, µCi/ml	(A) 8.20 Eo	8.24 E-2	1.28 Eo	9.27 E-3	7.92 E-4	2.14 E-3
I-131/I-133	1.6800	.8165	1.0900	1.6502	.3189	.7665
Hydrogen, cc/kg	46.7	20.8	30.8	42.8	31.1	35.2
Lithium, ppm (D)	1.10	.40	.76	1.62	1.05	1.29
Boron-10, ppm +	113.97	57.43	68.21	127.40	115.25	122.89
Oxygen-16, ppm	(E) .005	.000	.000	.000	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	7.12	6.54	6.85	6.99	6.64	6.74

+ Boron-10 = Total Boron x 0.396

NON-RADIOACTIVE CHEMICAL RELEASES, POUNDS
T.S. 4.13.A.6

Phosphate 0.0 (F) Boron 564 (F)

Sulfate 0.0 (F) Chromate 0.0 (F)

50% NaOH 0.0 (F) Chlorine (F) 0.0

REMARKS: (A) High activity level following reactor trip 8/24; (B) PC-4 not in service used NC-2; (C) Hydrogen level too low (following reactor trip); recommended increased pressure on VCT. (D) Four separate lithium additions 8/6, 8/10, 8/17, & 8/27; (E) following reactor trip; (F) The levels of these chemicals should create no adverse environmental impact.

SURRY POWER STATION

CHEMISTRY REPORT

Sept. 19 82

T.S. 6.6.3.d

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., µCi/ml	3.85^{0}	2.49^{0}	2.98^{0}	(A) 5.01^{-1}	1.78^{-1}	2.67^{-1}
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, µCi/ml	2.92^{-1}	8.64^{-2}	1.58^{-1}	2.01^{-1}	1.71^{-1}	1.90^{-1}
Iodine-131, µCi/ml	2.10^{-1}	6.76^{-2}	9.73^{-2}	4.91^{-3}	9.83^{-4}	2.25^{-3}
I-131/I-133	1.8218	.5867	.9837	1.6538	.3442	.8066
Hydrogen, cc/kg	32.2	(B) 24.6	28.6	39.9	25.6	32.0
Lithium, ppm (C)	1.20	.67	.87	1.45	.97	1.22
Boron-10, ppm +	57.04	41.55	49.03	117.40	103.10	109.90
Oxygen-16, ppm	.000	.000	.000	.000	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	7.17	6.90	7.05	6.85	6.62	6.72

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL
RELEASES, POUNDS
T.S. 4.13.A.6

Phosphate (D) Boron 772. (D)

Sulfate (D) Chromate 0.0 (D)

50% NaOH (D) Chlorine (D)

REMARKS: (A) High activity level due to secured let-down flow(for 40 hrs) prior to this sample. (B) Unable to sample hot leg for 10 days prior to this sample; recommended increase in VCT pressure. (C) Five separate lithium additions unit 1; two separate addtions unit 2. (D) The levels of these chemicals should create no adverse environmental impact.

DESCRIPTION OF ALL INSTANCES WHERE THERMAL DISCHARGE LIMITS WERE EXCEEDED

September, 1982

Due to the impairment of the circulating water system on the following days, the thermal discharge limits were exceeded as noted.

September 1 - Exceeded 15°F ΔT across station*
2 - Exceeded 15°F ΔT across station*
3 - Exceeded 15°F ΔT across station*
4 - Exceeded 15°F ΔT across station*
5 - Exceeded 15°F ΔT across station*
7 -- Exceeded 15°F ΔT across station*
12 - Exceeded 15°F ΔT across station*
13 - Exceeded 15°F ΔT across station
14 - Exceeded 15°F ΔT across station*
15 - Exceeded 15°F ΔT across station*
16 - Exceeded 15°F ΔT across station*
18 - Exceeded 15°F ΔT across station*
19 - Exceeded 15°F ΔT across station
20 - Exceeded 15°F ΔT across station
21 - Exceeded 15°F ΔT across station
22 - Exceeded 15°F ΔT across station
23 - Exceeded 15°F ΔT across station*
24 - Exceeded 15°F ΔT across station*
25 - Exceeded 15°F ΔT across station*
26 - Exceeded 15°F ΔT across station*
28 - Exceeded 17.5 F ΔT across station*
29 - Exceeded 15°F ΔT across station
30 - Exceeded 15°F ΔT across station

*Indicates dates where station ΔT was less than or equal to 15.0°F across station for some time during the day.

These ΔT excursions were allowable under Technical Specification 4.14.B.2. There were no reported instances of adverse environmental impact.

FUEL HANDLING

September, 1982

The originally installed cask drop pad, designed to absorb the impact of a shipping cask dropped in the Spent Fuel Pit, was removed September 28, 1982 in preparation for the installation of a cask drop pad of more modern design.

UNIT 1
SEPTEMBER, 1982

FUEL HANDLING

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL
	NONE DURING THIS REPORTING PERIOD.		

FUEL HANDLING

DATE SHIPPED/RECEIVED	NO. OF ASSEMBLIES PER SHIPMENT	ANSI NO. INITIAL ENRICHMENT	NEW OR SPENT FUEL SHIPPING CASK ACTIVITY LEVEL

NONE DURING THIS REPORTING PERIOD.

PROCEDURE REVISIONS THAT CHANGED THE OPERATING MODE DESCRIBED IN THE FSAR

SEPTEMBER, 1982

None during this reporting period.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

SEPTEMBER - 1982

PT-18.4 Boron Injection Tank Level Check - was not completed as scheduled on 7-27-82 for Unit 2. This PT is to verify that the boron injection tank remains full during plant operations. Station deviation report S2-82-219 was submitted and the test was completed satisfactorily on 8-1-82.

PT-2.27 Core Cooling Monitor - was not completed as scheduled on 8-5-82 for Unit 2. This PT provides a step by step functional test of the Core Cooling Monitor. Station deviation report S2-82-235 was submitted, and the test was completed satisfactorily on 8-18-82.

PT-2.26 (P-1-403) (P-1-458) Reactor Coolant System Pressure (Functional Test) and Testing of PORV (Unit 1) This test is to periodically stroke valves to monitor proper valve operation. MOV-1535 and MOV-1536 were not cycled within 90 days as specified in this test and station deviation report S1-82-322 was submitted. On 6-19-82 the valves were cycled.

PT-16.5 Containment Personnel Air Lock Test (Unit 1) - was performed 10-4-81 and scheduled to be performed again 4-4-82. However, the test was not performed within the ± 25% grace period, and station deviation report S1-82-291 was submitted. The test was completed satisfactorily on 6-4-82.

INSERVICE INSPECTION

SEPTEMBER, 1982

None during this reporting period.

REPORTABLE OCCURRENCES PERTAINING TO ANY OUTAGE OR POWER REDUCTIONS

SEPTEMBER, 1982

None during this reporting period.

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

MECHANICAL MAINTENANCE

UNIT2-82/10/01
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RECEIVED	SYS	COMP	MARKNO	SUMMARY	WKDESC	U	MR	DOCUMENTS
09/06/82	MS	VALVE	2-MS-268	REPAIR FURMANITED BODY TO BONNET LEA	VOID COMPLETED ON MRS206151505	2	207220829	0
09/06/82	MS	VALVE	2-MS-268	REPAIR TO ORIGINAL FURMANITED	VOID COMPLETED ON MRS206150800	2	207221827	0
DEPT TOTAL								0

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

MECHANICAL MAINTENANCE

SEPTEMBER, 1982

None during this reporting period.

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

ELECTRICAL MAINTENANCE

UNIT2-82/10/01
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	WR	TOTIMNEW
09/05/82	IA	MOTOR	2-IA-C-4A	MOTOR NOT START EVEN THOUGH THE INPT	RESET BREAKER	2	209050639	5
09/05/82	SW	MOV	MOV-SW-202A	MOV IS GROUNDED	DRIED OUT MOTOR TESTED SAT	2	209051200	2
09/06/82	CH	MOTOR	2-CH-P-1A	AUX OIL PUMP THERMALS SWAP 2B MTR TO	REPLACEMENT MOTOR INSTALLED	2	209060645	0
								--
DEPT TOTAL								7

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

ELECTRICAL MAINTENANCE

September, 1982

None during this reporting period.

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 1

INSTRUMENT MAINTENANCE

UNIT2-82/10/01
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETURNDATE	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	WR	DOWNTIME
09/06/82	RI	ALARM	A-A-8	ALARM IS IN FOR NO REASON	RESET MAG AMP TO CLEAR ALARM	2	209061209	0
DEPT TOTAL								0

MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS

UNIT NO. 2

INSTRUMENT MAINTENANCE

September, 1982

None during this reporting period.

HEALTH PHYSICS

SEPTEMBER, 1982

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CRF20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

PROCEDURE NO.	UNIT	TITLE	DATE DEVIATED	DATE SNSOC REVIEWED
PT 2.1 (T-1-432)	1	Overpower - Overtemperature Delta T Protection (T-1-412, 422, 432)	08/18/82	09/02/82