

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
 UNIT Davis-Besse Unit 1
 DATE November 9, 1981
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 259-5000,
Ext. 251

MONTH October, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	845
2	845
3	878
4	883
5	879
6	877
7	883
8	880
9	886
10	878
11	868
12	880
13	882
14	881
15	878
16	864

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	36
19	273
20	615
21	802
22	432
23	308
24	663
25	803
26	844
27	844
28	833
29	822
30	767
31	770

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

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 Ext. 251

OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: October, 1981
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	745	7,296	36,605
12. Number Of Hours Reactor Was Critical	716.1	5,111.3	19,495.5
13. Reactor Reserve Shutdown Hours	19.7	444.5 424.8	5,306.9 3,326.6
14. Hours Generator On-Line	686.8	4,911.3	17,959.1
15. Unit Reserve Shutdown Hours	0	0	1,731.4
16. Gross Thermal Energy Generated (MWH)	1,746,360	11,698,556	38,603,362
17. Gross Electrical Energy Generated (MWH)	577,925	3,896,036	12,871,370
18. Net Electrical Energy Generated (MWH)	545,363	3,656,100	11,920,601
19. Unit Service Factor	92.2	67.3	49.8
20. Unit Availability Factor	92.2	67.3	54.9
21. Unit Capacity Factor (Using MDC Net)	82.3	56.3	38.4
22. Unit Capacity Factor (Using DER Net)	80.8	55.3	37.7
23. Unit Forced Outage Rate	7.8	28.2	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1982 Refueling Outage Scheduled February 26, 1982 - May 21, 1982

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346
 UNIT NAME Davis-Besse Unit 1
 DATE November 9, 1981

REPORT MONTH October, 1981

COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 259-5000, Ext. 2

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
15	81 10 16	F	38.5	A	3	NA	NA	NA	The reactor tripped on high Reactor Coolant System pressure.
16	81 10 22	F	19.7	A	2	NA	NA	NA	The turbine was taken off-line following CIV problems. The reactor was manually tripped following a Steam and Feedwater Rupture Control System trip. See Operational Summary for further details.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from Previous Month
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

OPERATIONAL SUMMARY

- 10/1/81 - 10/16/81 Reactor power was maintained at approximately 100% full power with the generator gross load at approximately 915 MWe until 2334 hours on October 16, 1981, when the reactor tripped on high Reactor Coolant System (RCS) pressure. The trip was due to a loss of non-essential Bus F-3. Bus F-3 was lost because of a ground fault sensed by transformer BF-3. As a result, generator stator cooling water and main feedwater block valve indication were lost which led to the trip.
- 10/17/81 - 10/18/81 The reactor was critical at 1830 hours. The turbine generator was synchronized on line at 1403 hours on October 18, 1981.
- 10/19/81 - 10/22/81 The reactor power was slowly increased and was limited by having No. 1 High Pressure Feedwater Heater string out of service. The unit reached approximately 95% power on October 21, 1981. Reactor power was maintained at 95% until 1100 hours on October 22, 1981 when the operator initiated a manual reduction of reactor power due to combined intercept valves (CIVs) 3 and 4 problems.
- The reactor power was maintained at approximately 8% with the turbine off-line until 1519 hours on October 22, 1981, when the reactor was manually tripped following a Steam and Feedwater Rupture Control System trip.
- When the turbine was reset to get hydraulic pressure for CIV testing, steam pressure control was switched from steam generator outlet to steam header indication. Due to slight differences between these pressure indications, turbine bypass valves started closing which eventually led to a low steam generator level causing the Steam and Feedwater Rupture Control System trip.
- 10/23/81 The reactor was critical at 0119 hours. The turbine generator was synchronized on line at 1019 hours.
- 10/24/81 - 10/31/81 Reactor power was slowly increased and attained 97% power on October 29, 1981. Reactor power was reduced to 94% due to a leaking extraction steam bellows expansion joint in #1 Low Pressure Turbine to #1 Deaerator.
- Reactor power was maintained at approximately 94% for the remainder of the month.

REFUELING INFORMATION

DATE: October, 1981

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: February 26, 1982
3. Scheduled date for restart following refueling: May 21, 1982
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Reload analysis is scheduled for completion as of December, 1981. No technical specification changes or other license amendments identified to date.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. February 1982
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None identified to date

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
(a) 177 (b) 44 - Spent Fuel Assemblies
8 - New Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
Present 735 Increase size by 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
Date 1988 - assuming ability to unload the entire core into the spent fuel pool is maintained