AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse Unit 1
DATE August 7, 1981
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000, Ext. 251

MONTH July, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	74
4	0	20	368
5	0	21	658
6	0	22	852
7	0	23	872
8	0	24	876
9	0	25	871
10	0	26	868
11	0	27	878
12	0	28	870
13	0	29	875
14	0	30	297
15	0	31	0
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-346
DATE August 7, 1981
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TELEPHONE (419) 259-5000, Ext. 251

OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: July, 1981
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,087	34,396
12. Number Of Hours Reactor Was Critical	271.4	3,297	17,681.2
13. Reactor Reserve Shutdown Hours	39.8	74.6	2,956.7
14. Hours Generator On-Line	267.6	3,146.6	16,194.4
15. Unit Reserve Shutdown Hours	0	0	1,731.4
16. Gross Thermal Energy Generated (MWH)	642,660	7,034,943	33,939,749
17. Gross Electrical Energy Generated (MWH)	212,015	2,350,398	11,325,732
18. Net Electrical Energy Generated (MWH)	193,927	2,196,512	10,461,013
19. Unit Service Factor	36.0	61.9	47.8
20. Unit Availability Factor	36.0	61.9	53.1
21. Unit Capacity Factor (Using MDC Net)	29.3	48.5	35.9
22. Unit Capacity Factor (Using DER Net)	28.8	47.7	35.3
23. Unit Forced Outage Rate	64.0	32.0	26.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: August 14, 1981
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit 1
DATE August 7, 1981
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000, Ext. 25

REPORT MONTH July, 1981

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12 Cont	81 06 24	F	436.6	H	3	NP-33-81-44	EB	CTBRT	The reactor tripped due to a loss of E2 power during control rod drive breaker logic surveillance testing. Loss of E2 was due to mechanical shock from construction in the area.
13	81 07 30	F	39.8	A	3	NA	NA	NA	The reactor tripped on reactor protection system (RPS) low reactor coolant system (RCS) pressure. See operational summary for further details.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation
5-Reduction
6-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

OPERATIONAL SUMMARY
JULY, 1981

7/1/81 - 7/19/81 The unit remained shutdown following the reactor trip on June 24, 1981 to repair the reactor coolant pump (RCP) seals and investigate the failure of control rod 8 in group 5.

Three RCP seals have been removed, refurbished, and reinstalled.

The cause of the failure of control rod 8 in group 5 to respond to withdrawal commands was determined to be a broken leaf spring lodged in the control rod drive mechanism.

7/19/81 The reactor was critical at 0048 hours. The turbine-generator was synchronized on line at 0439 hours.

7/20/81 - 7/30/81 The reactor power was slowly increased and attained 100% power on July 22, 1981. Reactor power was maintained at 100% full power until July 30, 1981 when the boot seal between the low pressure turbine and high pressure condenser started to leak causing an increase in condenser pressure. When condenser vacuum could not be maintained, the operator initiated a manual reduction of reactor power. This rapid power decrease caused Tave and RCS pressure to drop causing a RPS trip on low RCS pressure which tripped the reactor at 14% power at 0813 hours.

The unit remained shutdown to repair the boot seal.

REFUELING INFORMATION

DATE: July, 1981

1. Name of facility: Davis-Besse Unit 1

2. Scheduled date for next refueling shutdown: March, 1982

3. Scheduled date for restart following refueling: May, 1982

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Reload analysis is scheduled for completion as of December, 1981. No technical specification changes or other license amendments identified to date.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. January, 1982

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None identified to date

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 44 - Spent Fuel Assemblies
8 - New Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present 735 Increase size by 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date 1988 - (assuming ability to unload the entire core into the spent fuel pool is maintained)