8211110339 821013 PDR ADOCK 05000213 R PDR SEPTEMBER 1982

FOR THE MONTH OF

MONTHLY OPERATING REPORT NO. 82-9

CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM, CONNECTICUT

HADDAM NECK PLANT

#### PLANT OPERATIONS

The following is a summary of plant operations for the month of September, 1982:

The unit operated at 100% power until September 12 when load was reduced for a short period of time to conduct routine turbine stop valve testing.

A load reduction to remove the plant from service was commenced on September 17 at 1630 hours to correct main transformer moisture inleakage. While the unit was off line, the B station battery was replaced and maintenance was performed on the No. 1 main steam trip valve.

The unit initially phased on line September 25 at 1908 but the turbine tripped at 2011 (auto stop oil trip). Following the completion of turbine trip tests while off line, the turbine tripped again at 2141.

After verifying simulated turbine overspeed trip setpoint, the unit was phased on line at 2316 on September 25, and reached full power at 0237 on September 27.

The plant reduced power for a short period of time on September 28 to plug condenser tube leaks. The unit remained at full power throughout the rest of the month of September.

SYSTEM	I&C September Rep MALFU	port for 1982 NCTION	EFFECT ON SAFE	ORRECTIVE ACTION TAKEN TO PREVENT	TAKEN TO PROVIDE FOR REACTOR SAFETY	
COMPONENT	CAUSE	RESULT	OPERATION	REPETITION	DURING REPAIR .	
4 Feedwater Regula- ting Valve Control Solenoid (SOV)	Resilient seats stick due to being energized for long periods of time	Valve should not close when turbine tripped	NONE	Ordering special SOVs designed for long term steady state operations	NONE	
Master Cycler	Excessive use of Master Cycler causing a relay to hang up	Unable to drive rods	NONE	Re-wet the mercury wetted relay contacts	NONE	
Steam Line Break #3 Indicator	Normal end of life of interior component (slide wire)	Unreliable steam line break indi- cation. Redun- dant data avail- able from data logger	NONE	Replaced slidewire	Inserted trip signal into loop #3 steam line break	

SYSTEM OR	Maintenance Repo MALFU	rt for Sept. 1982 NCTION	ON SAFE	CORRECTIVE ACTION TAKEN TO PREVENT	TAKEN TO PROVIDE FOR REACTOR SAFETY	
COMPONENT	CAUSE	RESULT	OPERATION	REPETITION	DURING REPAIR	
#1 Main Steam trip valve	Tail link to link bore not concentric with rock shaft.	Valve disk would not trav- el to full open during valve test.	NONE	Valve disassembled and tail link was repaired	N/A	
	Reason: Tail link was distorted from prior maint.					
#B Battery	Leaking cell <u>Reason</u> : Age	Inoperative battery	NONE .	Installed new battery	Reactor in hot standby	
"A" Aux. Feed pump steam control valve	Leakage past valve seat <u>Reason</u> : Normal wear	Excessive leakage thru valve	NONE	Rebuilt valve	N/A	
#4 Feed Reg. Valve	Leakage past valve seat <u>Reason</u> : Valve plug cracked	Valve would not function proper- ly	NONE	Installed new plug & stem	N/A	

\*\*\*\*\*\*\*NRC OPERATING STATUS REPORT COMPLETED BY REACTOR ENGINEERING\*\*\*\*\*\*\*

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1. 2. 3. 4. 5. 6. 7. 3. 9. 10.	DOCKET50-213 O P E R A T I N G REPORTING PERIODSEPTEMBER 1982 UTILITY CONTACTBOB EPPINGER (203 LICENSED THERMAL POWER(MNT)BOB EPPINGER (203 LICENSED THERMAL POWER(MNT)BOB EPPINGER (203 DESIGN ELECTRICAL RATING(NET MWE)	S T A T U S OUTAGE + ON-LINE HR ) 267-2556 EX274 1825 .67 X 0.9 = 600.3 582 582 555 FE REASONS N/A MWE) N/A	195.1 + 524.9 = 720.0	**************************************	
11.	REASON FOR RESTRICTION. IF ANY	N/A			-
5		MONTH	YR. TO DATE	CUMULATIVE TO DATE	
12	HOURS IN REPORTING PERIOD	720.0	6551.0	129287 0 *	
13.	NUMBER OF HOURS THE REACTOR WAS CRITICAL	720.0	6510.0	112061.5 *	
14.	REACTOR RESERVE SHUTDOWN HOURS	195.1	446.0	1638.5 *	
15.	HOURS GENERATOR ON LINE	524.9	6038.7	• 106941.7 *	
16.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	373.7	
17.	CASSS THERMAL ENERGY GENERATED (MWH)	916859.	10721960.	185371216.	
18.	GROSS ELECTRICAL ENERGY GENERATED (MWH)	296932.	3514015.	60987492.	
19.	NET ELECTRICAL ENERGY GENERATED (MWH)	280707.	3345447.	57925899.	
20.	UNIT SERVICE FACTOR	72.9	92.2	82.7 *	
21.	UNIT AVAILABILITY FACTOR	72.9	92.2	83.0 *	
22.	UNIT CAPACITY FACTOR (USING MDC NET)	70.2	92.0	82.5 *	
23.	UNIT CAPACITY FACTOR (USING DER NET)	67.0	87.7	76.3 *	
24.	UNIT FORCED OUTAGE RATE	27.1	7.8	6.6 *	
25.	UNIT FORCED OUTAGE HOURS	195.1	512.3 *SINCE COMME	7570.6 * RCIAL OPERATION 1/1/68	

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH) ...

REFUELING ESTIMATED FOR 1/22/83. ESTIMATED DURATION- 7 WEEKS.

27. IF CURRENTLY SHUTDOWN, ESTIMATED STARTUP DATE...... N/A

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### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-213 Conn. Yankee UNIT Haddam Neck

DATE October 13, 1982

COMPLETED BY S. Unikewicz

TELEPHONE (203) 267-2556

DAY	AVERAGE	DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1 <u>- 1 - 1 - 1</u>	564	17	427
2		564	. 8	0
3		564	19	0
4		565	20	0
5		565	21	0
6		562	22	0
7		562	23	0
8		562	24	0
9		56	25	0
10		564	26	217
11		562	27	562
12		536	28	463
13		562	29	570
14		561	30	571
15		560	31	
16		560		

### INSTRUCTIONS

MONTH: September 1982

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Complete the nearest whole megawatt.

(9/77)

# CONNECTICUT YANKEE REACTOR COOLANT DATA MONTH: SEPTEMBER 1982

	REACTOR COOLANT ANALYSIS	12.00	MINIMUM		AVERAGE		MAXIMUM	
				••		•••		
	PH @ 25 DEGREES C	:	6.14E+00		6.55E+00	*	6.86E+C0 :	
	CONDUCTIVITY (UMHOS/CM)	:	4.30E+00	:	7.32E+00	:	1.07E+01 :	
-	CHLORIDES (PPM)	:	<5.00E-02	:	<5.00E-02	:	<5.00E-02 :	
	DISSOLVED OXYGEN (PPB)	:	<5.00E+00	:	<5.00E+00	:	<5.00E+00 :	
	BORON (PPM)	:	2.80E+02	:	4.16E+02	:	7.89E+02 :	
1	LITHIUM (PPM)	:	3.90E-01	:	7.07E-01	:	9.20E-01 :	
	TOTAL GAMMA ACT. (UC/ML)	:	6.50E-02	:	1.07E+00	:	1.75E+00 :	
	IDDINE-131 ACT. (UC/ML)	:	1.61E-03	:	7.42E-03	:	9.62E-03 :	
-	I-131/I-133 RATIO	:	6.08E-01	:	8.01E-01	:	9.53E-01 :	
	CRUD (MG/LITER)	:	<1.00E-02	:	<1.00E-02	:	<1.00E-02 :	
	TRITIUM (UC/ML)		5.82E-01	:	1.62E+00	:	2.32E+00 :	
-	HYDROGEN (CC/KG)	:	5.44E+00	:	1.86E+01	:	3.45E+01 :	
		• • • • •	• • • • • • • • • • • •	••	• • • • • • • • • • • •	•••	• • • • • • • • • • • • • •	

AERATED LIQUID WASTE PROCESSED(GALLCNS):	1.18E+05
WASTE LIQUID PROCESSED THROUGH BORON RECOVERY (GALLONS):	1.20E+05
AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE):	1.15E+00
PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE):	0.00E+00

# UNIT SHUTDOWNS AND POWER REDUCTIONS

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DOCKET NO. <u>50-213</u> UNIT NAME <u>Conn. Yankee</u> DATE <u>October 13</u> COMPLETED BY <u>S. Unikewicz</u> TELEPHONK203) 267-2556

REPORT MONTH September 1982

No.	Date	Type <sup>1</sup>	Duration (Hours)	. Reason <sup>2</sup>	Method of Shutting 3 Down Reactor	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
	820917	F	195	В	I .	N/A	EB	TRANSF	Moisture in main transformer oil. Filtered oil, re- paired leaks, tested and returned to ser- vice.
					••	LER 82-06/3L	EC	BATTRY	Battery internal plates swelled caus- ing stress on battery casing resulting in cracking of casing. Replaced battery.
	820928	N/A	0	A	N/A	N/A	N/A	N/A	Reduced power to plug condenser tubes

1 F Forced S Scheduled	2 Reason: A-Equipment Failure(Explain) B-Maintenance or Test C-Refueling D-Regulatory Restriction	H-Other(Explain)	3 Method: 1-Manual 2-Manual Scram 3-Automatic Scram. 4-Other(Explain	Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report(LER)File (NUREG-0161)	
1.	E-Operator Training&License Ex F-Administrative C-Operational Error(Explain	amination		5 Exhibit 1 Same Source	

### REFUELING INFORMATION REQUEST

# 1. Name of facility

Connecticut Yankee Atomic Power Company

2. Scheduled date for next refueling shutdown.

'January 22, 1983

3. Scheduled date for restart following refueling.

Approximately six to eight weeks.

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

No technical specification changes are anticipated at this time.

- (b) If answer is yes, what, in general, will these be?
- N/A
- (c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the .core reload (Ref. 10 CFR Section 50.59)?
- (d) If no such review has taken place, when is it scheduled? N/A
- Scheduled date(s) for submitting proposed licensing action and supporting information.

There are no scheduled dated because of (4) above.

Important licensing considerations associated with refueling,
e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 157 (b) 389

- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies. 1168
- The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1994 to 1995