

## OPERATING DATA REPORT

**DOCKET NO.** 50-344  
**DATE** 10-6-82  
**COMPLETED BY** W.O. Nicholson  
**TELEPHONE** (503) 556-3713  
 Extension-409

### OPERATING STATUS

1. Unit Name: TROJAN NUCLEAR PLANT  
 2. Reporting Period: September, 1982  
 3. Licensed Thermal Power (MWt): 3411  
 4. Nameplate Rating (Gross MWe): 1216  
 5. Design Electrical Rating (Net MWe): 1130  
 6. Maximum Dependable Capacity (Gross MWe): 1122  
 7. Maximum Dependable Capacity (Net MWe): 1080  
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes - Correction: August, 1982 "Unit Shutdowns and Power Reductions" form should show outage 82-06 duration of 574.1 instead of 485 hours.

NA

9. Power Level To Which Restricted, If Any (Net MWe): NA  
 10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	6,551	53,303
12. Number Of Hours Reactor Was Critical	679.2	2,832.0	32,295.2
13. Reactor Reserve Shutdown Hours	0.0	571.3	2,743.1
14. Hours Generator On-Line	642.4	2,697.4	31,185.5
15. Unit Reserve Shutdown Hours	0.0	571.3	2,080.0
16. Gross Thermal Energy Generated (MWH)	2,107,909	8,616,242	98,135,018
17. Gross Electrical Energy Generated (MWH)	690,849	2,766,139	31,935,510
18. Net Electrical Energy Generated (MWH)	655,380	2,594,172	30,101,861
19. Unit Service Factor	89.2	41.2	58.5
20. Unit Availability Factor	89.2	49.9	62.4
21. Unit Capacity Factor (Using MDC Net)	84.3	36.7	52.3
22. Unit Capacity Factor (Using DER Net)	80.6	35.0	50.0
23. Unit Forced Outage Rate	5.3	6.0	20.6
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	NA		

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA  
 26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

	Forecast	Achieved
	NA	NA
	NA	NA
	NA	NA

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-344  
 UNIT Trojan  
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MONTH SEPTEMBER, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1042</u>	17	<u>862</u>
2	<u>1029</u>	18	<u>1057</u>
3	<u>1081</u>	19	<u>1062</u>
4	<u>1078</u>	20	<u>1065</u>
5	<u>1069</u>	21	<u>1065</u>
6	<u>91</u>	22	<u>1068</u>
7	<u>30</u>	23	<u>1063</u>
8	<u>269</u>	24	<u>1064</u>
9	<u>20</u>	25	<u>1065</u>
10	<u>820</u>	26	<u>1067</u>
11	<u>1081</u>	27	<u>1069</u>
12	<u>1077</u>	28	<u>1071</u>
13	<u>1079</u>	29	<u>1069</u>
14	<u>868</u>	30	<u>1064</u>
15	<u>902</u>	31	<u>NA</u>
16	<u>1062</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

**UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-344  
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REPORT MONTH SEPTEMBER 1982

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
82-07	820906	S	41.6	B	2	NA	NA	NA	Scheduled outage to repair a body-to-bonnet leak on RCS RTD manifold hot leg outlet isolation valve, 8073C. On the power reduction the turbine was manually tripped at 10% power resulting in a reactor trip.
82-08	820908	F	30.1	A	1	NA	NA	NA	The reactor was manually tripped at 54% power due to a trip of the south MFP. A faulty pressure switch on the pump lube oil system was replaced. The plant remained off the line to perform maintenance on various secondary system components.
82-09	820914	F	3.0	G	3	NA	NA	NA	The reactor tripped from S/G 'C' low level with low feedwater flow due to a north main feedwater pump trip. I&C technicians inadvertently caused a ground on instrument bus Y02 during troubleshooting of the generator core monitor. Since the north MFP governor is also powered from Y02, the pump subsequently tripped.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

**UNIT SHUTDOWNS AND POWER REDUCTIONS**

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REPORT MONTH Sept, 1982

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No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
82-10	820917	F	2.9	A	3	NA	NA	NA	The reactor tripped on Lo-Lo S/G level due to a south main feedwater pump trip. The south MFP trip was caused by a failed proximitor in the thrust bearing wear detector. The proximitor was replaced.

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SUMMARY OF OPERATING EXPERIENCE

DOCKET NO: 50-344  
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OPERATION: The plant entered September operating at 100% power. On Sept 1 at 0600 a Radiological Emergency Response Plan unusual event was declared due to greater than 1 gpm unidentified reactor coolant system leakage calculated by a periodic test. The excessive leakage calculation was subsequently determined to be erroneous and the unusual event was canceled at 0955 on Sept 1.

A scheduled plant shutdown to Mode 3 was commenced at 0200 Sept 6 to repair a body-to-bonnet leak on valve 8073C on the reactor coolant loop 'C' RTD manifold hot leg outlet isolation valve. The valve gasket was replaced and the plant was returned to power operation at 2028 on Sept 7.

The reactor was manually tripped from 54% power due to a trip of the south MFP on Sept 8. A faulty pressure switch on the south MFP lube oil system was replaced. The plant was returned to service on Sept 9 after completing repairs on various secondary system components. (see attached sheet for continuation)

**MAJOR SAFETY RELATED MAINTENANCE:**

A gasket was replaced on reactor coolant loop 'C' RTD manifold hot leg outlet isolation valve 8073C to stop a body-to-bonnet leak. The spare service water pump was removed and disassembled to perform its routine three-year inspection. The lower radial bearing has been replaced. The inspection was still in progress at the end of September. Work began in September on the containment purge exhaust system modification to strengthen the exhaust ducting.

MISCELLANEOUS MAINTENANCE: Completed the inspection and plugging of tubes in 'C' condenser and feedwater heaters 2BA and 2BB. Completed replacement of eroded pipe in 6B feedwater heater drain line. Completed rebuilding the east spent fuel pool cooling pump and the north reactor coolant drain tank pump. Completed seal replacement on the north boric acid transfer pump. Replaced damaged components in the south MFP thrust bearing wear detector and removed the shims on the thrust bearing. Replaced diaphragms on two valves which were leaking by their seats in the boric acid evaporator system. Completed repair of the north heater drain tank relief valves. Continued construction work on the control building workspace.

LICENSE CHANGES: NONE

MISCELLANEOUS: A Radiological Emergency Response Plan training exercise was performed on Sept 16.

On September 14 a reactor trip occurred from 'C' steam generator low level with low feedwater flow due to a north MFP trip. Instrumentation and Control Technicians troubleshooting a problem on the generator core monitor inadvertently caused a ground on nonpreferred 120-volt AC instrument bus Y02. Since the north MFP governor is also powered from Y02, the pump subsequently tripped. The plant was returned to service within three hours.

The reactor tripped on lo-lo steam generator level following a south MFP trip on September 17. The pump tripped due to a failed proximator in the thrust bearing wear detector. The proximator was replaced and the plant was returned to service within three hours.

The plant operated at or near 100% power for the remainder of the month. Slight power reductions were made on six days during September due to high condenser back pressure.

The reactor coolant system dose equivalent iodine level remained fairly constant throughout the month at approximately 0.3 uCi/ml. The primary to secondary leak rate was less than 1 gal per day during September.