



Northern States Power Company

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January 7, 1991

ASME Code Section XI IWA-6220

Director
Office of Nuclear Reactor Regulation
Attn: Document Control Desk
US Nuclear Regulatory Commission

PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket No. 50-222 License No. DPR-42 50-306 DPR-60

Unit 2 Inservice Inspection Summary Report
Refueling Outage No. 14

Attached for your information are four copies of the report, "Inservice Inspection Examination Summary, August 29, 1990 to October 2, 1990, Refueling Outage Number 14, Inspection Period 2, Second Interval" and four copies of the ASME Section XI Repair/Replacement Summary report. The Inservice Inspection Examination Summary report is a summary of the examinations performed during the last refueling outage of the Prairie Island Nuclear Generating Plant Unit No. 2. The ASME Section XI Repair/Replacement Summary report is a summary of the repairs and replacements to ASME Code Class 1, 2 and 3 components and systems during the last refueling outage of the Prairie Island Nuclear Generating Plant Unit No. 2.

The Inservice Inspection Summary report identifies components examined, the examination methods used, the examination number, and summaries the results. The examination plan focused on 1) the pressure retaining components and their supports of the reactor coolant system and associated systems classified as ASME Class 1 and ASME Class 2, 2) FSAR Augmented Examinations, and 3) Eddy current examination of the Steam Generator. All anomalies were either corrected or an engineering evaluation was performed to accept "as in" conditions. The ASME Section XI Repair/Replacement Summary report identifies work request number and summaries the work.

The Inservice Inspection Summary Report is being submitted in accordance with the Prairie Island ASME Code Section XI Inservice Inspection and Testing Program. The submittals of the attached summary reports are intended to satisfy the inspection reporting requirements contained in IWA-6220 of the ASME Boiler and Pressure Vessel Code.

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Please contact us if you have any questions related to the information presented in these reports.

Thomas M Parker

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Manager

Nuclear Support Services

c: Regional Administrator-III, NRC (2 copies of attachments) NRR Project Manager, NRC (1 copy of attachment)

NRC Resident Inspector (w/o attachment)

J Silberg (w/o attachment)

A Justin, Chief Boiler Inspector, State of MN (1 copy of attachment)

S Tack, Hartford Ins. (1 copy of attachment)

Attachment

SP

#### TITLE:

## ASME SECTION XI REPAIR/REPLACEMENT SUMMARY

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### 1.0 PURPOSE

This surveillance activity is performed to review work requests for the purpose of documenting all ASME repairs/replacements. The details of the repairs/replacements are found in the individual work packages. This SP activity will result in a summary of the repairs/replacements and meets the requirements of N1ACD 5.4.

6.4.5 which states "a summary report **SHALL** be submitted to Manager Nuclear Support Services subsequent to each outage for all repairs/replacements conducted since the previous outage."

## 2.0 SCOPE OF REVIEW

- 2.1 The review and summary description is limited to repairs/replacements governed by ASME Section XI.
- 2.2 The review and summary description of repairs are:
  - 2.2.1 Those made to the pressure retaining boundary of components and their supports which involve metal removal or welding when no new base material is added.
  - 2.2.2 Heat exchanger tube or tube sheet bore hole plugging.
  - 2.2.3 Welds joining replacements to existing systems.
- 2.3 The review and summary of the modifications or alterations are:
  - 2.3.1 Spare and renewal components, appurtenances, subassemblies, or parts of a component or system.
  - 2.3.2 Additions of components such as valves that are within the scope of ASME Section XI.
  - 2.3.3 System changes such as rerouting of piping that are within the scope of ASME Section XI.

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## 3.0 INSTRUCTIONS

- 3.1 Following review of the work instructions (WRs), make entries onto the attached table.
- 3.2 Following the completion of start of the unit after refueling, submit table to Manager Nuclear Technical Services.

WR #	SYSTEM	DESCRIPTION OF THE WORK
P7524-FW-Q	FW	GRIND/BUFF OUT INDICATION ON LINE 16-2FW-13
P7215-MS-Q	MS	BUFF OUT INDICATION ON MAIN STEAM LINE
P7559-MS-Q	MS	BUFF OUT INDICATION ON LINE 30-2MS-1
P7601-RH-Q	RH	BUFF OUT INDICATION ON LINE 8-2RC-15B
P7247-SG-Q	SG	BUFF OUT INDICATION AT WELD E ON 21 SG
P7507-SG-Q	SG	GRINDOUT INDICATION ON 22 SG OUTLT NOZZLE
P7386-SI-Q	SI	BUFF OUT WELD #2591 ON 22 SIP PPG
P7357-ZC-Q	ZC ZC	REPAIR 24 FCU DRAIN PPG
P7391-CC-Q	cc	REPAIR WELD ON 21 CC HX MANWAY

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### REPLACEMENTS

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WR #	SYSTEM	DESCRIPTION OF THE WORK
P4193-CC-Q	CL	REPLACE 21 CC HX OUTLET CL SPOOLPIECE
P0099-CL-Q	CL	REPLACE 2CL-43-1, 21 CL PMP DISCH CK
P0098-CL-Q	CL	REPLACE 2CL-39-1, 21 CL PMP DISCH ISOL
P0011-RC-Q	RC	REPLACE 2RC-7-2, LOOP A SPRY B/P VLV
P6985-RC-Q	RC	REPLACE 2RC-10-2, PZR SAFETY VLV
P5755-EM-Q	EM	INSTALL VLVS AT RVLIS 2LE 752, 762
P5745-RC-Q	RC	REPLACE ISOLATION MANIFOLD 2PT-419
P6534-ZC-Q	2C	REPLACE FCU CL RELIEF VLV
P3513-CS-Q	CS	INSTALL RECIRC LINE, 21 CS PMP
P3514-CS-Q	CS	INSTALL RECIRC LINE, 22 CS PMP
P4045-CC-Q	cc	REPLACE 21 CC HX CHANNEL MANWAY
P5434-VC-Q	VC	INSTALL DRAIN VALVES ON 21, 22 RCP SEAL IN-
P5448-ZC-Q	zc	INSTALL 21/23 FCU ANNUBARS
P5449-ZC-Q	ZC	INSTALL 22/24 FCU ANNUBARS
P5584-AF-Q	AF	INSTALL 12/22 AFWP FLUSH TEES
P5668-SI-Q	SI	INSTALL 21 SI PUMP MINI FLOW OR)F

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### REPLACEMENTS

REPAIR/REPLACEMENT SUMMARY			
WR #	SYSTEM	DESCRIPTION OF THE WORK	
P5669-SI-Q	SI	INSTALL 22 SI PUMP MINI FLOW ORIF	
P7310-ZC-Q	ZC	REPLACE LEAKING H-BENDS ON 22/24 FCU	
P7530-ZC-Q	ZC ZC	REPLACE LEAKING H-BENDS ON 21/23 FCU	
P6597-ZC-Q	ZC	RMV/RPL 21/23 FCU RELIEF VLVS	

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