



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

MAINE YANKEE ATOMIC POWER COMPANY

DOCKET NO. 50-309

MAINE YANKEE ATOMIC POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118  
License No. DPR-36

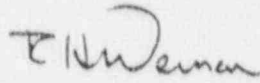
1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for amendment (emergency situation) by Maine Yankee Atomic Power Company (the licensee) dated December 28, 1990, and supplemented on December 31, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.B.6.(b) of Facility Operating License No. DPR-36 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard Wessman, Director  
Project Directorate I-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Charges to the Technical  
Specifications

Date of Issuance: January 4, 1991



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ATTACHMENT TO LICENSE AMENDMENT NO. 118

FACILITY OPERATING LICENSE NO. DPR-36

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Replace the following page of the Appendix A Technical Specifications with the attached page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

4.10-5 (Table 4.10-2)

Insert

4.10-5 (Table 4.10-2)

TABLE 4.10-2  
STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required*	Result	Action Required*	Result	Action Required*
A minimum of 5% tubes per S.G.	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 25% tubes in this S.G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect additional 45% tubes in this S.G.	C-1	None
					C-2	Plug defective tubes
					C-3	Perform action for C-3 result of first sample
			C-3	Perform action for C-3 result of first sample	N/A	N/A
			All other S.G.s are C-1	None	N/A	N/A
			Some S.G.s C-2 but no additional S.G. are C-3	Perform action for C-2 result of second sample	N/A	N/A
			Additional	Inspect all tubes in each S.G. and plug defective tubes.  Report to NRC pursuant to 10 CFR 50.72(b)(2)	N/A	N/A
		C-3	Inspect all tubes in this S.G., plug defective tubes and inspect 25% tubes each S.G.  Report to NRC pursuant to 10 CFR 50.72(b)(2)			

S =  $3 \frac{N}{n}$  Percent where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection

\*NOTE: For the inspection of steam generator #1 performed in December 1990, a 100% inspection of steam generator #2 rows 3 through 10 shall be conducted instead of the additional inspections of S/G #1 indicated by this specification. If the inspection of #2 steam generator indicates any u-bend defects, then a 100% inspection of #3 steam generator rows 3 through 10 shall be conducted.